

December 31, 2022

Docket No. 52-050

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
One White Flint North  
11555 Rockville Pike  
Rockville, MD 20852-2738

**SUBJECT:** NuScale Power, LLC Submittal of the NuScale Standard Design Approval Application - Part 9, "Withheld Information," Revision 0

**REFERENCES:**

1. NuScale letter to NRC, "NuScale Power, LLC Submittal of Planned Standard Design Approval Application Content," dated February 24, 2020 (ML20055E565)
2. NuScale letter to NRC, "NuScale Power, LLC Requests the NRC staff to conduct a pre-application readiness assessment of the draft, 'NuScale Standard Design Approval Application (SDAA),' " dated May 25, 2022 (ML22145A460)
3. NRC letter to NuScale, "Preapplication Readiness Assessment Report of the NuScale Power, LLC Standard Design Approval Draft Application," Office of Nuclear Reactor Regulation dated November 15, 2022 (ML22305A518)
4. NuScale letter to NRC, "NuScale Power, LLC Staged Submittal of Planned Standard Design Approval Application," dated November 21, 2022 (ML22325A349)

NuScale Power, LLC (NuScale) is pleased to submit Part 9 of the Standard Design Approval Application (SDAA), "Withheld Information," Revision 0, described in Reference 1. NuScale submits the part in accordance with requirements of 10 CFR 52 Subpart E, Standard Design Approvals. As described in Reference 4, the enclosure is part of a staged SDAA submittal. NuScale requests NRC review, approval, and granting of standard design approval for the US460 standard plant design.

From July 25, 2022 to October 26, 2022, the NRC performed a pre-application readiness assessment of available portions of the draft NuScale Final Safety Analysis Report (FSAR) to determine the FSAR's readiness for submittal and for subsequent review by NRC staff (References 2 and 3). Part 9 was not included in the scope of the readiness assessment.

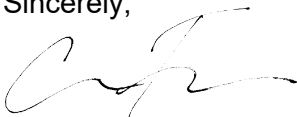
Enclosure 1 contains SDAA Part 9, "Withheld Information," Revision 0.

This letter makes no regulatory commitments and no revisions to any existing regulatory commitments.

If you have any questions, please contact Mark Shaver at 541-360-0630 or at [mshaver@nuscalspower.com](mailto:mshaver@nuscalspower.com).

I declare under penalty of perjury that the foregoing is true and correct. Executed on December 31, 2022.

Sincerely,



Carrie Fosaaen  
Senior Director, Regulatory Affairs  
NuScale Power, LLC

Distribution: Brian Smith, NRC  
Michael Dudek, NRC  
Getachew Tesfaye, NRC  
Bruce Bovol, NRC  
David Drucker, NRC

Enclosure 1: SDAA Part 9, "Withheld Information," Revision 0

**Enclosure 1:**

SDAA Part 9, "Withheld Information," Revision 0



NuScale US460 Plant  
Standard Design Approval Application

---

# Withheld Information

---

## PART 9

Revision 0

©2022, NuScale Power LLC. All Rights Reserved

---

## COPYRIGHT NOTICE

This document bears a NuScale Power, LLC, copyright notice. No right to disclose, use, or copy any of the information in this document, other than by the U.S. Nuclear Regulatory Commission (NRC), is authorized without the express, written permission of NuScale Power, LLC.

The NRC is permitted to make the number of copies of the information contained in these reports needed for its internal use in connection with generic and plant-specific reviews and approvals, as well as the issuance, denial, amendment, transfer, renewal, modification, suspension, revocation, or violation of a license, permit, order, or regulation subject to the requirements of 10 CFR 2.390 regarding restrictions on public disclosure to the extent such information has been identified as proprietary by NuScale Power, LLC, copyright protection notwithstanding.

Regarding nonproprietary versions of these reports, the NRC is permitted to make the number of additional copies necessary to provide copies for public viewing in appropriate docket files in public document rooms in Washington, DC, and elsewhere as may be required by NRC regulations. Copies made by the NRC must include this copyright notice in all instances and the proprietary notice if the original was identified as proprietary.

---

**Part 9 - Withheld Information**

Part 9 of the NuScale Power, LLC, Standard Design Approval Application identifies the location of security-related information within the Final Safety Analysis Report (FSAR).

NuScale requests the security-related information be withheld from public disclosure in accordance with 10 CFR 2.390(d)(1).

Security-related information in the following areas is withheld.

<b>FSAR Location</b>	<b>Description</b>
Figure 1.2-8	Reactor Building 25'-0" Elevation
Figure 1.2-9	Reactor Building 40'-0" Elevation
Figure 1.2-10	Reactor Building 55'-0" Elevation
Figure 1.2-11	Reactor Building 70'-0" Elevation
Figure 1.2-12	Reactor Building 85'-0" Elevation
Figure 1.2-13	Reactor Building 100'-0" Elevation
Figure 1.2-14	Reactor Building 126'-0" Elevation
Figure 1.2-15	Reactor Building 146'-6" Elevation
Figure 1.2-16	Reactor Building East-West Section View
Figure 1.2-17	Reactor Building North-South Section View
Figure 1.2-18	Control Building 100'-0" Elevation
Figure 1.2-19	Control Building 125'-0" Elevation
Figure 1.2-20	Control Building North-South Section View
Figure 1.2-21	Control Building East-West Section View
Figure 1.2-22	Radioactive Waste Building 70'-0" Elevation
Figure 1.2-23	Radioactive Waste Building 82'-0" Elevation
Figure 1.2-24	Radioactive Waste Building 100'-0" Elevation
Figure 1.2-25	Radioactive Waste Building 119'-0" Elevation
Figure 1.2-26	Radioactive Waste Building 145'-0" Elevation
Figure 1.2-27	Radioactive Waste Building North-South Section View
Figure 1.2-28	Radioactive Waste Building East-West Section View
Table 3.4-1	Limiting Flooding Sources and Maximum Steady-State Flood Heights
Table 3.4-2	Flood Levels for Rooms Containing Systems, Structures, and Components Subject to Flood Protection (Without Mitigation)
Figure 9.1.3-2	Ultimate Heat Sink Water Level and Plant Feature Elevations
Section 9A.3.8	Manual Fire Suppression
Section 9A.5	Fire Hazards Analysis
Table 9A-8	Reactor Building Fire Areas
Table 9A-9	Radioactive Waste Building Fire Areas
Table 9A-10	Control Building Fire Areas
Figure 12.3-1a	Reactor Building Radiation Zone Map - 25' Elevation
Figure 12.3-1b	Reactor Building Radiation Zone Map - 40' Elevation
Figure 12.3-1c	Reactor Building Radiation Zone Map - 55' Elevation
Figure 12.3-1d	Reactor Building Radiation Zone Map - 70' Elevation
Figure 12.3-1e	Reactor Building Radiation Zone Map - 85' Elevation
Figure 12.3-1f	Reactor Building Radiation Zone Map - 100' Elevation
Figure 12.3-1g	Reactor Building Radiation Zone Map - 126' Elevation
Figure 12.3-1h	Reactor Building Radiation Zone Map - 145' 6" Elevation
Figure 12.3-2a	Radioactive Waste Building Radiation Zone Map - 70' Elevation
Figure 12.3-2b	Radioactive Waste Building Radiation Zone Map - 82' Elevation

***Withheld Information***

---

<b>FSAR Location</b>	<b>Description</b>
Figure 12.3-2c	Radioactive Waste Building Radiation Zone Map - 100' Elevation
Figure 19.5-1	General Arrangement Reactor Building Equipment Door