## **APPENDIX C - ABBREVIATIONS**

Aac Amps alternating current
AAPS auxiliary ac power source
ABS auxiliary boiler system
AC acceptance criteria
ac alternating current

ACI American Concrete Institute

ACRS Advisory Committee on Reactor Safeguards

ADAMS Agencywide Documents Access and Management System

AEA Atomic Energy Act
AFT as-found tolerance
AFU air filtration unit

AFWS automatic auxiliary feedwater system
AFWST auxiliary feedwater storage tank

AHU air handling unit

AIA Aircraft Impact Assessment

AICC adiabatic isochoric with complete combustion

AIL accident-induced leakage

AISC American Institute of Steel Construction

AISI American Iron and Steel Institute

AL analytical limit

ALARA as low as reasonably achievable

ALI annual limits on intake

ALT as-left tolerance

AMS Aerospace Material Specification

ANB Annex Building

ANBVS Annex Building HVAC system
ANL Argonne National Laboratory
ALWR advanced light-water reactor
ANS American Nuclear Society

ANSI American National Standards Institute

AO axial offset

AOAS axial offset anomaly

AOO anticipated operational occurrence

AOV air-operated valve

APL actuation and priority logic

APRM average power range [neutron flux] monitor

AQ-S as augmented quality

Ar argon

AR acoustic resonance

ASAI application-specific action item
ASCE American Society of Civil Engineers

ASHRAE American Society of Heating, Refrigerating, and Air-Conditioning

Engineers

ASME American Society of Mechanical Engineers
ASTM American Society for Testing and Materials

ATWS anticipated transient without scram

AWS American Welding Society

B boron Ba Barium

BACE boric acid corrosion evaluation

BAS boron addition system
BDBE beyond-design-basis event

BDBEE beyond-design-basis external event

BDG backup diesel generator

BISI bypassed and inoperable status indication

BL bulletin

BOC beginning of cycle BIST built-in self-test

BNL Brookhaven National Laboratory
BPDS balance-of-plant drain system
BPSS backup power supply system
BPV Boiler and Pressure Vessel

BPVC Boiler and Pressure Vessel Code

BTP branch technical position
BWR boiling-water reactor

C carbon C Celsius

cal/g calories per gram

CARS condenser air removal system

CAS compressed air system

CB control building CBA cost-benefit analysis

CBP computer-based procedures

CCF common-cause failure

CCFP conditional containment failure probability

CCTV closed-circuit television system

CDE core damage event
CDF core damage frequency

CDI conceptual design information CDST core damage source term

CEAC control element assembly calculator
CES containment evacuation system

CET containment event tree

CFD computational fluid dynamics

CFD containment flood and drain

CFDS containment flooding and drain system

cfm cubic feet per minute

CFR Code of Federal Regulations

CFT containment flange tool

CFWS condensate and feedwater system

CHF critical heat flux CHFR critical heat flux ratio

CHRS containment heat removal system(s)

Ci/yr curies per year

CILRT containment integrated leak rate test

CIS containment isolation system
CIV containment isolation valve

CLD complex logic device CLL collapsed liquid level

cm centimeter(s)

CM communication module cmh cubic meters per hour CMTR certified material test report

CNTS containment system
CNV containment vessel
CNV containment volume
CoF Coefficient of friction
COL combined license

COLR core operating limits report
COMS communications systems
ConOps concept of operations
COT channel operational test
CP construction permit
CPC core protection calculator

CPC core protection calculator
CPS condensate polishing system

Cr-Mo chrome-molybdenum CRA control rod assembly

CRAGT control rod assembly guide tube

CRAM containment response analysis methodology

CRB control building

CRC cyclic redundancy check

CRD control rod drive

CRDM control rod drive mechanism

CRDS control rod drive system

CREATCS Control Room Emergency Air Temperature Control System

CRE control room envelope

CRHS control room habitability system

CRVS control room heating, ventilation, and air conditioning system

CRVS control room ventilation system

CS core support

CSA core support assembly

CSDRS certified seismic design response spectra

CSDRS-HF CSDRS-high frequency
CSF critical safety function
CST condensate storage tank
CTB calibration and testing bus
CUF cumulative usage factor

CVAP Comprehensive Vibration Assessment Program

CVCS Chemical and Volume Control System

CVCSI CVCS Isolation

CWS circulating water system

DAC derived air concentration
DAC design acceptance criteria
DBA design-basis accident

DBAST Design Basis Analysis Source Term

DBE design-basis event

DBFFF design-basis failed fuel fraction

DBPB design basis pipe break
DBST design basis source term

DBT design-basis threat
DBT design basis tornado
DC design certification
dc direct current

DCA design certification application
DCD design control document
DCS distributed control system

DE dose equivalent

DF decontamination factor
DGB diesel generator building
DHR decay heat removal

DHRS decay heat removal system
DI design implementation

DID defense-in-depth (also refers to as D3)

DLF dynamic load factor
DM direct method

DNBR departure from nucleate boiling ratio

DO dissolved oxygen

DOD Department of Defense DOE Department of Energy

DOT Department of Transportation

DPR Division of Preparedness and Response

D/Q deposition factor

DR decay ratio

D-RAP design reliability assurance program
DSRS design specific review standard

DWO density wave oscillation
DWS demineralized water system

DWSI demineralized water system isolation
DWST demineralized water storage tank
DWT demineralized water treatment

EAB exclusion area boundary

EAF environmentally assisted fatigue ECCS emergency core cooling system EDG emergency diesel generator

EDNS (refers to NuScale) normal dc power system

EDSS (refers to NuScale) highly reliable dc power system

EDSS-C EDSS-common

EFDS equipment and floor drain system

EFPDS effective full power day

EFPYs full-power years

EHVS switchyard system (Chapter 8)
EIM equipment interface module

ELVS electrical distribution system (Chapter 8)

EM evaluation model

EMI electromagnetic interference

EMVS electrical distribution system (Chapter 8)

ENDF Evaluated Nuclear Data File

EOC end of cycle

EOF emergency operations facility

EOL End-of-life

EOP emergency operating procedure

EP emergency planning

EPAs electrical penetration assemblies
EPA Environmental Protection Agency
EPRI Electric Power Research Institute

EQ equipment qualification

EQRF equipment qualification record file
ERDS emergency response data system
ERG emergency response guideline
ERS equipment requirement specification

ESF engineered safety feature

ESFAS engineered safety features actuation system

ESM extended subtraction method

ESS-MS module specific

ETAP Electrical Transient Analyzer Program

E-W east-west

F Fahrenheit

FA function allocation

FAC flow-accelerated corrosion

FCU fan coil unit FE finite element

FEA finite element analysis
FEI fluid-elastic instability
FEM finite element model

FEMA failure modes and effects analysis
F<sub>en</sub> factor for environmental fatigue

FFD fitness for duty
F/G flutter and galloping
FHA fire hazards analysis
FHA fuel handling accident
FHE fuel handling equipment
FHM fuel handling machine

FIRS foundation input response spectra

FIV flow-induced vibration

FMEA failure modes and effects analysis

FOAK first-of-a-kind factor of safety

FPGA field programmable gate array

FPP fire protection program

FPRA fire probabilistic risk assessment

FPS fire protection system  $F_Q$  flux hot channel factor FR Federal Register

FRA functional requirements analysis

FRA/FA functional requirements analysis and function allocation

FRED EPRI Fuel Reliability Database FSAR final safety analysis report FSER final safety evaluation report FSI fluid structure interaction

FSSD fire safe shutdown

ft Feet or foot FTR final test report

FWIV feedwater isolation valve
FWLB feedwater line break
FWPD feedwater pipe break
FWRV feedwater regulating valve
FWRV feedwater regulation valve

FWS feedwater system

g grams

GAC Granulated Activated Charcoal
GALE gaseous and liquid effluent
GALL generic aging lessons learned
GCB generator circuit breaker

Gd<sub>2</sub>O<sub>3</sub> gadolinium oxide

GDC general design criterion/criteria

GE General Electric
GL generic letter

GMRS ground motion response spectrum

gph gallon per hour gpm gallons per minute

GRWS gaseous radioactive waste system

GSI generic safety issue

GTG generic technical guidance GTS generic technical specification

Gy/h grays per hour

H₂ hydrogen H-3 tritium hr hour

HA hazard analysis
HA human action
HCSG helical coil SG

HCSG helical coil steam generator
HCW high-conductivity waste
HDCI high duty core index
HDPE high-density polyethylene
HDR Heissdampf reactor

HED human engineering discrepancy

HELB high energy line break

HEPA high-efficiency particulate air

HF high frequency

HFE human factors engineering

HFEITS human factors engineering issues tracking system

HFP hot full power

HIC high integrity container

HICR highly-integrated control rooms—human factors issue

HIPS highly integrated protection system

HIPS TR highly integrated protection system topical report

HMI human machine interface
HOV hydraulic operated valve
HPS Health Physics Society
HPU hydraulic power unit
HRA human reliability analysis
HSI human system interfaces

HTP heat transfer plate

HTP™ High Temperature Performance

HVAC heating, ventilation, and air conditioning

HWM hard-wired module

Hz Hertz

HZP hot zero power

l iodine

I&C instrumentation and control IAB inadvertent actuation block

IAEA International Atomic Energy Agency

IAS instrument air system

IASCC irradiation-assisted stress corrosion cracking

ICC inadequate core cooling ICI in-core instrumentation

ICIGT in core instrumentation guide tube ICIS in-core instrumentation system

ICSBEP International Handbook of Evaluated Criticality Safety Benchmark

Experiments

IDDS interface design descriptions

IE infrequent event

IE inspection and enforcement

IEEE Institute of Electrical and Electronic Engineers IFPRA internal flooding probabilistic risk assessment

IGSCC intergranular stress-corrosion cracking

IHAS identify important human action ILRT integrated leakage rate testing

IN information notice

in. inch(es)

INL Idaho National Laboratory

I/O input/output

IODHRS inadvertent operation of the DHRS

IP implementation plan IP inspection procedure

iPWR integral pressurized-water reactorIRM information and records managementIRPI individual rod position indicator system

ISG interim staff guidance ISI inservice inspection

ISLOCA interfacing system loss-of-coolant accident

ISM independent support motion ISRS in-structure response spectra

ISS in structure shock IST inservice testing

ISV integrated system validation

ITA inspections, tests, and analyses

ITAAC Inspections, tests, analyses, and acceptance criteria

ITP initial test program

IV&V independent verification and validation

K Kelvin

KA knowledge and abilities kg/s kilograms per second

km kilometer(s) kPa kilopascals

KSA knowledge, skills, and abilities/aptitude

ksf kilopound per square foot ksi kilopound per square inch

kV kilovolt

LAR large release frequency

LBB leak-before-break lbm pound-mass

lbs pounds

lb/ft pounds per square-foot

LCO limiting condition for operation

LCS local control station

LCW low-conductivity waste

LFI leakage flow instability

LHGR linear heat generation rate

I/m liters per minute
LLRT local leak-rate test

LLRW low-level radioactive waste LOCA loss-of-coolant accident LOCV loss of condenser vacuum

LOEL loss of external load LOFW limiting loss of feedwater LOOP loss-of-offsite power

LPMS loose parts monitoring system
LPSD low power shutdown operations

LPZ low population zone
LRF large release frequency

LRWS liquid radioactive waste system LSSS limiting safety system setting

LTOP low temperature overpressure protection

LTSD long-term shutdown LTSP limiting trip setpoint

LUHS loss of normal access to the normal heat sink

LWA limited work authorization

LWR light-water reactor

m meter(s)

MACCS MELCOR Accident Consequence Code System

MC Main condenser

MCHFR minimum critical heat flux ratio

MCNP Monte Carlo N-Particle Transport Code

MCR main control room MCS module control system

MG main generator

MHS module heatup system

MIB monitoring and indication bus
MIP Measurement and Inspection Plan

MIR module inspection rack
MLA module lift adapter
MLA module lifting adapter
MLD master logic diagram

mm millimeters

MMAF multi-module adjustment factor MMBtu million British Thermal Units

MOC middle of cycle

MOV motor-operated valve

MPa megapascals mph miles per hour

MPS module protection system MPT main power transformer

MR maintenance rule

Mrem millirem

MRP materials reliability program

MRRS minimum required response spectrum

m/s meters per second

MS main steam

MSLB main steam line break

MSIBV main steam isolation bypass valves

MSIV main steam isolation valve

MSL main steam lines

MSO multiple spurious operation
MSPB main steam pipe break
MSS main steam system

MSSD minimum safe standoff distance

MSSV main steam safety valve

mSv milliSievert

MTC moderator temperature coefficient

MW megawatt

MWS maintenance work station

MWt megawatts thermal

N neutron N-16 Nitrogen-16

NDE nondestructive examination
NDS nitrogen distribution system
NDT nil ductility temperature
NEI Nuclear Energy Institute

NEMA National Electrical Manufacturers Association

NFJC new fuel jib crane

NFPA National Fire Protection Association

NiCrFe Nickel-chromium-ion

NIST NuScale Integral System Test Facility

NMS neutron monitoring system

NOPD normal operating pressure differential

NPM nuclear power modules NPM NuScale Power Module

NPP nuclear power plant NPS nominal pipe size

NQA nuclear quality assurance

NRC Nuclear Regulatory Commission

NRELAP5 NuScale Reactor Excursion and Leak Analysis Program, Version 5

NRR Office of Nuclear Reactor Regulation

N-S north-south

NSIR Nuclear Security and Incident Response

NSSS nuclear steam supply system

NTSP nominal trip setpoint
NuFuel NuScale fuel assembly

NUMARC Nuclear Utilities Management and Resources Council

NUSCALE NuScale Power, LLC

O<sub>2</sub> oxygen

OBE operating-basis earthquake
ODCM offsite dose calculation manual
OER operating experience review

OHLHS overhead heavy-load handling systems

OI open items
OL operating license

OM Operation and Maintenance Code

OPV Over-pressurization Vent

ORE occupational radiation exposure
ORIGEN Oak Ridge Isotope GENeration
ORNL Oak Ridge National Laboratory's

OSC operational support center

OSHA Occupational Safety and Health Administration

OT□T overtemperature □T

P&ID piping and instrumentation diagram

P<sub>a</sub> psia

PA protected area
PA postulated accident
PAM Post-accident monitoring

PASS post-accident sampling system
PAT performance assessment testing
pcm/s percent millirho per second
PCP process control program
PCS plant control system

PCT peak cladding temperature

PCUS pool cleanup system
PDC principal design criterion

PDILs power-dependent insertion limits

PERMISS process and effluent radiation monitoring instrumentation and

sampling system

PGA peak ground acceleration PHT pressurizer heater trip

PIRT phenomena identification and ranking table

PIV pressure isolation valve

PLDS pool leakage detection system

PLS plant lighting system
PMF probable maximum flood

PMWP probable maximum winter precipitation
PNNL Pacific Northwest National Laboratory

POC proof of concept
POS plant operating state
POV power-operated valve
ppm parts per million

PPS plant protection system

PRA probabilistic risk assessment
PRHA pipe rupture hazards analysis
P-STG plant specific technical guideline

PSCIV primary system containment isolation valve

PSCS pool surge control system
psf pounds per square-foot
PSI preservice inspection
psi pounds per square inch

psia pounds per square-inch absolute psig pound per square inch gauge PSS physical security system PSS process sampling system PST phase separator tanks PST preservice testing

PSWS potable and sanitary water system

PTAC performance and test acceptance criteria PTLR pressure and temperature limits report

P-T pressure-temperature
PTS pressurized thermal shock
PWR pressurized-water reactors
PWS potable water system

PWSCC primary water stress-corrosion cracking

PZR pressurizer

QA quality assurance

QAP quality assurance program

QAPD quality assurance program description

QG quality group

QHO quantitative health objective QMP quality management plan QPTR Quadrant Power Tilt Ratio

rad/h radiation absorbed doses per hour

RADTRAD RADionuclide Transport, Removal And Dose

RAI request for additional information

RAMP Radiation Protection Computer Code Analysis and Maintenance

Program

RAP reliability assurance program

RBC reactor building crane

RBVS reactor building heating ventilation and air conditioning system

RBVS reactor building HVAC system RBVS reactor building ventilation system

RBVS RXB ventilation system

RCCW reactor component cooling water

RCCWS reactor component cooling water system

RCPB reactor coolant pressure boundary

RCS reactor coolant system

RCSB reactor coolant pressure boundary

REA rod ejection accident

rem roentgen equivalent man (a unit of radiation dose)
REMP radiological environmental monitoring program
RETS radiological effluent technical specification

RFFF realistic failed fuel fraction

RFI radio frequency interference
RFP reactor pool and refueling pool

RFT reactor flange tool

RFT reactor vessel flange tool

RG regulatory guide

RHR residual heat removal
RIS regulatory issue summary
RITSTF Risk-Informed TSTF
RMS root mean square
RO reactor operator

**RPCS** reactor pool cooling system RPI rod position indication **RPS** reactor protection system **RPV** reactor pressure vessel RRS required response spectrum **RRV** reactor recirculation valve RSR results summary report **RSS** remote shutdown station **RSV** reactor safety valves **RTB** reactor trip breaker RTF reactor flange tool

RTM requirement traceability matrix

RT<sub>NDT</sub> reference temperature nil ductility temperature RTNSS regulatory treatment of non-safety systems

RTP rated thermal power RTS reactor trip system RV reactor vessel

RVI reactor vessel internal RVV reactor vent valve RWB radwaste building RWBVS RWB HVAC system

RWDS radioactive waste drain system RWS radioactive waste building

RXB reactor building

s second

SA situation awareness

SAFDL specified acceptable fuel design limit

SAM seismic anchor motion

SAPHIRE Systems Analysis Programs for Hands-on Integrated Reliability

**Evaluations** 

SAR safety analysis report SAS service air system

SASSI system for analysis of soil-structure interaction

SAT site acceptance test

SBAC smooth bounding analysis curve

SBEDS single degree of freedom blast design spreadsheet

SBM scheduling and bypass module

SBO station blackout

SBT scenario-based testing SC-1 Seismic Category I

SCALE Standardized Computer Analyses for Licensing Evaluation

SCC stress corrosion cracking SCWS site cooling water system

SBM safety data bus

SDD software design description

SDIS safety display and indication system
SDIS safety display and information system

SDM shutdown margin

SMA seismic margins analysis

SDOE secure development and operational environment

SE safety evaluation

SEI Structural Engineering Institute

SER safety evaluation report SFC single failure criterion

SFCP Surveillance Frequency Control Program
SFDP Safety Function Determination Program

SFM safety function module

SFP spent fuel pool

SFPC spent fuel pool cooling

SG safety group SG steam generator

SGI safeguards information

SGIFR steam generator tube inlet flow restrictor

SGS steam generator system
SGTF steam generator tube failure

SGTR SG tube rupture

SGTR steam generator tube rupture

SI International System
SIL software integrity level

SL safety limit SLB steam line break SM shift manage

SMA seismic margins analysis

SMACNA Sheet Metal and Air Conditioning Contractors' National Association

SME subject matter expert SMR small modular reactor

SNAP Symbolic Nuclear Analysis Package

SOARCA State-of-the-Art Reactor Consequence Analyses

SOC sampling of operational condition

SOV solenoid-operated valve

SP setpoint program

SPDS safety parameter display system SPND self-powered neutron detectors

SPV staffing plan validation SR surveillance requirement S&Q staffing and qualifications

SRM staff requirements memorandum SRWS solid radioactive waste system

SRO senior reactor operator SRP Standard Review Plan

SRS software requirements specification

SRSS square-root-sum-of-square SRST spent resin storage tank SRST spent resin storage tanks

SSC structure, system, and component

SSCIV secondary system containment isolation valve

SSE safe-shutdown earthquake SSI secondary system isolation SSI soil-structure interaction

SSSI structure-soil-structure interaction

Std Standard

STS standard technical specifications

Sv sievert

SVM scheduling and voting module

TA task analysis

TADOT TRIP ACTUATING DEVICE OPERATIONAL TEST

TASCS thermal stratification, cycling, and striping

TB turbulent buffeting
TBS turbine bypass system

TBVS turbine building ventilation system

Tc technetium

TCV turbine control valve

TEDE total effective dose equivalent

TeR technical report
TF transfer function

TGB turbine generator building
TGS turbine generator system
TGSS turbine gland seal system

TH thermal-hydraulic TID total integrated dose

TIHA treatment of important human actions

TLX task load index
TMI Three Mile Island
TNT Trinitrotoluene
TOM top of module

TP typical

TR technical report

TRACE TRAC/RELAP Advanced Computational Engine
TRITON Time-dependent Operation for Neutronic depletion

TRS test response spectrum

TRU Transuranic

TS technical specifications
TSC technical support center
TSS top support structure

TSTF Technical Specifications Task Force

TSV turbine stop valve TT thermally treated

TT turbine trip

UAT unit auxiliary transformer
UFC Uniform Facility Code
UHS ultimate heat sink

UL Underwriters Laboratories

UO<sub>2</sub> uranium dioxide URA upper riser assembly

URD Utility Requirements Document URS uniform response spectrum

US United States
USE upper shelf energy

USNRC United States Nuclear Regulatory Commission

USM uniform support motion

V volt

VBS vehicle barrier system
V/H vertical to horizontal
VHRA very high radiation area

VLA vented lead-acid

VRLA valve-regulated lead-acid

VS vortex shedding VT visual testing

V&V verification & validation

WRC Welding Research Council

WSW wet solid waste Wt% weight percent

Xe xenon

χ/Q atmospheric dispersion factor

ZOI zone of influence

ZPA zero period acceleration