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Item No.	**DESCRIPTION**	**ATTACHMENT**	**SECTION and TITLE**
I. Feedwater Flow Measurement Technique and Power Measurement Uncertainty			
I.1.	Detailed description of plant-specific implementation of feedwater flow measurement technique and power increase gained as a result of implementing technique	Attachment 1	3.1 Background and General Approach 3.2 LEFM Ultrasonic Flow Measurement and Core Thermal Power Uncertainty
I.1.A.	NRC approval of topical report on flow measurement technique	Attachment 1	3.2.1 LEFM Flow Measurement
I.1.B.	Reference to NRC's approval of proposed measurement technique	Attachment 1	3.2.1 LEFM Flow Measurement
I.1.C.	Plant Implementation	Attachment 1	3.2.2 Plant Implementation
I.1.D.	Disposition of NRC criteria	Attachment 1	3.2.4 Disposition of NRC Criteria for Use of Topical Reports
I.1.E.	Total power measurement uncertainty calculation for the plant	Attachment 1 Attachment 11	3.2.3 LEFM and Core Thermal Power Measurement Uncertainty and Methodology Core Thermal Power Uncertainty Calculation
I.1.F.	Calibration and maintenance	Attachment 1	3.2.4 Disposition of NRC Criteria for Use of Topical Reports 3.2.5 Deficiencies and Corrective Actions
I.1.G.	Proposed outage time for LEFM and basis for selected time	Attachment 1	3.2.4 Disposition of NRC Criteria for Use of Topical Reports
I.1.H	Proposed actions if outage time is exceeded, and basis for actions	Attachment 1	3.2.4 Disposition of NRC Criteria for Use of Topical Reports
II. Accidents and Transients for which the Existing Analyses of Record Bound Plant Operation at the Proposed Uprated Power Level			
II.1	Matrix for bounded accidents and transients	Attachment 6	9.0 Reactor Safety Performance Evaluations

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Item No.	**DESCRIPTION**	**ATTACHMENT**	**SECTION and TITLE**
III. Accidents and Transients for which the Existing Analyses of Record Do Not Bound Plant Operation At the Proposed Uprated Power Level			
III.1,2, 3	Matrix for unbounded accidents and transients	Attachment 6	9.0 Reactor Safety Performance Evaluations
IV. Mechanical/Structural/Material Component Integrity and Design			
IV.1.A.i	Reactor vessel, nozzles, supports	Attachment 6	3.2 Reactor Vessel 3.2.1 Fracture Toughness 3.2.2 Reactor Vessel Structural Evaluation
IV.1.A.ii	Reactor core support structures and vessel internals	Attachment 6 Attachment 1	3.3 Reactor Internals 3.3.1 Reactor Internal Pressure Difference 3.3.2 Reactor Internals Structural Evaluation 3.3.3 Steam Separator and Dryer Performance 3.4 Flow-Induced Vibration 3.4.2 Adverse Flow Effects
IV.1.A.iii	Control rod drive mechanisms	Attachment 6	2.5 Reactivity Control

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Item No.	**DESCRIPTION**	**ATTACHMENT**	**SECTION and TITLE**
IV.1.A.iv	Nuclear Steam Supply System (NSSS) piping, pipe supports, branch nozzles	Attachment 6	3.4 Flow-Induced Vibration 3.5 Piping Evaluation 3.5.1 Reactor Coolant Pressure Boundary Piping 3.6 Reactor Recirculation System 3.7 Main Steam Line Flow Restrictors 3.8 Main Steam Isolation Valves 3.9 Reactor Core Isolation Cooling 3.10 Residual Heat Removal System 3.11 Reactor Water Cleanup System
IV.1.A.v	Balance of plant (BOP) piping (NSSS interface systems, safety-related cooling water systems, and containment systems)	Attachment 6	3.5 Piping Evaluation 3.5.2 Balance-of-Plant Piping Evaluation 6.4.1 Service Water Systems 6.4.3 Reactor Building Closed Cooling Water System
IV.1.A.vi	Steam generator tubes, secondary side internal support structures, shell and nozzles	NA	NA
IV.1.A.vii	Reactor coolant pumps	NA	NA
IV.1A.viii	Pressurizer shell, nozzles, and surge lines	NA	NA

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Item No.	**DESCRIPTION**	**ATTACHMENT**	**SECTION and TITLE**
IV.1.A.ix	Safety-related valves	Attachment 6	3.1 Nuclear System Pressure Relief/Overpressure Protection 3.8 Main Steam Isolation Valves 4.1 Containment System Performance 4.1.1 Generic Letter 89-10 Program 4.1.2 Generic Letter 95-07 Program 6.5 Standby Liquid Control System
IV.1.B.i	Stresses	Attachment 6	3.2 Reactor Vessel 3.2.2 Reactor Vessel Structural Evaluation 3.4 Flow-Induced Vibration 3.5 Piping Evaluation 3.5.1 Reactor Coolant Pressure Boundary Piping 3.5.2 Balance-of-Plant Piping Evaluation
IV.1.B.ii	Cumulative usage factors	Attachment 6	3.2.2 Reactor Vessel Structural Evaluation
IV.1.B.iii	Flow induced vibration	Attachment 6 Attachment 1	3.4 Flow-Induced Vibration 3.4.2 Adverse Flow Effects
IV.1.B.iv	Changes in temperature (pre- and post- uprate)	Attachment 6	1.3 TPO Plant Operating Conditions 1.3.1 Reactor Heat Balance 1.3.2 Reactor Performance Improvement Features Table 1-2 Thermal-Hydraulic Parameters at TPO Uprate Conditions

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Item No.	**DESCRIPTION**	**ATTACHMENT**	**SECTION and TITLE**
IV.1.B.v	Changes in pressure (pre- and post- uprate)	Attachment 6	1.3 TPO Plant Operating Conditions 1.3.1 Reactor Heat Balance 1.3.2 Reactor Performance Improvement Features Table 1-2 Thermal-Hydraulic Parameters at TPO Uprate Conditions
IV.1.B.vi	Changes in flow rate (pre- and post-uprate)	Attachment 6	1.3 TPO Plant Operating Conditions 1.3.1 Reactor Heat Balance 1.3.2 Reactor Performance Improvement Features Table 1-2 Thermal-Hydraulic Parameters at TPO Uprate Conditions
IV.1.B.vii	High-energy line break locations	Attachment 6	10.1 High Energy Line Break 10.1.1 Steam Line Breaks 10.1.2 Liquid Line Breaks
IV.1.B.viii	Jet impingement and thrust forces	Attachment 6	10.1 High Energy Line Break 10.1.1 Steam Line Breaks 10.1.2 Liquid Line Breaks 10.1.2.6 Pipe Whip and Jet Impingement
IV.1.C.i	Reactor vessel pressurized thermal shock calculations	Attachment 6	3.1 Nuclear System Pressure Relief/Overpressure Protection
IV.1.C.ii	Reactor vessel fluence evaluation	Attachment 6	3.2 Reactor Vessel 3.2.1 Fracture Toughness

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Item No.	**DESCRIPTION**	**ATTACHMENT**	**SECTION and TITLE**
IV.1.C.iii	Reactor vessel heatup and cooldown pressure-temperature limit curves	Attachment 6	3.2 Reactor Vessel 3.2.1 Fracture Toughness
IV.1.C.iv	Reactor vessel low temperature overpressure protection	Attachment 6	3.2 Reactor Vessel 3.2.1 Fracture Toughness
IV.1.C.v	Reactor vessel upper shelf energy	Attachment 6	3.2 Reactor Vessel 3.2.1 Fracture Toughness
IV.1.C.vi	Reactor vessel surveillance capsule withdrawal schedule	Attachment 6	3.2 Reactor Vessel 3.2.1 Fracture Toughness
IV.1.D	Code of record	Attachment 6	3.2 Reactor Vessel 3.2.2 Reactor Vessel Structural Evaluation 3.5 Piping Evaluation 3.5.1 Reactor Coolant Pressure Boundary Piping 3.5.2 Balance-of-Plant Piping Evaluation
IV.1.E	Component inspection/testing programs and erosion/corrosion programs	Attachment 6	3.5 Piping Evaluation 3.5.1 Reactor Coolant Pressure Boundary Piping 3.5.2 Balance-of- Plant Piping Evaluation 10.6 Plant Life
IV.1.F	NRC Bulletin 88-02, “Rapidly Propagating Fatigue Cracks in Steam Generator Tubes”	NA	NA
V. Electrical Equipment Design			
V.1.A	Emergency diesel generators	Attachment 6	6.1 AC Power 6.1.2 On-Site Power

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Item No.	**DESCRIPTION**	**ATTACHMENT**	**SECTION and TITLE**
V.1.B	Station blackout equipment	Attachment 6	9.3.2 Station Blackout
V.1.C	Environmental qualification of electrical equipment	Attachment 6	10.3 Environmental Qualification 10.3.1 Electrical Equipment
V.1.D	Grid stability	Attachment 6	6.1 AC Power 6.1.1 Off-Site Power
		Attachment 12	All Grid Stability Study
		VI. System Design	
VI.1.A	NSSS Interface Systems for BWRs (e.g., suppression pool cooling, as applicable)	Attachment 6	3.4 Flow-Induced Vibration 3.5 Piping Evaluation 3.5.1 Reactor Coolant Pressure Boundary Piping 3.5.2 Balance-of-Plant Piping Evaluation 3.6 Reactor Recirculation System 3.7 Main Steam Line Flow Restrictors 3.8 Main Steam Isolation Valves 3.9 Reactor Core Isolation Cooling 3.10 Residual Heat Removal System 3.11 Reactor Water Cleanup System
VI.1.B	Containment Systems	Attachment 6	4.1 Containment System Performance 4.1.1 Generic Letter 89-10 Program 4.1.2 Generic Letter 95-07 Program 4.1.3 Generic Letter 96-06

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Item No.	**DESCRIPTION**	**ATTACHMENT**	**SECTION and TITLE**
VI.1.C	Safety-related cooling water systems	Attachment 6	6.4 Water Systems 6.4.1 Service Water Systems 6.4.5 Ultimate Heat Sink
VI.1.D	Spent fuel pool storage and cooling systems	Attachment 6	6.3 Fuel Pool 6.3.1 Fuel Pool Cooling 6.3.2 Crud Activity and Corrosion Products 6.3.3 Radiation Levels 6.3.4 Fuel Racks
VI.1.E	Radioactive waste systems	Attachment 6	4.5 Standby Gas Treatment System 8.1 Liquid and Solid Waste Management 8.2 Gaseous Waste Management 8.3 Radiation Sources in the Reactor Core 8.4 Radiation Sources in Reactor Coolant 8.4.1 Coolant Activation Products 8.4.2 Activated Corrosion Products 8.4.3 Fission Products 8.5 Radiation Levels 8.6 Normal Operation Off-Site Doses

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Item No.	**DESCRIPTION**	**ATTACHMENT**	**SECTION and TITLE**
VI.1.F	Engineered safety features (ESF) heating, ventilation, and air conditioning systems	Attachment 6	4.4 Main Control Room Atmosphere Control System 4.7 Post-LOCA Combustible Gas Control System 6.6 Power Dependent Heating, Ventilation and Air Conditioning
VII. Other			
VII.1	Operator actions and effects on time available	Attachment 6	4.1 Containment System Performance 6.7 Fire Protection 9.3 Special Events 9.3.2 Station Blackout 10.5 Operator Training and Human Factors
VII.2.A	Emergency and abnormal operating procedures	Attachment 6	10.9 Emergency Operating Procedures
VII.2.B	Control room controls, displays (including the safety parameter display system) and alarms	Attachment 1 Attachment 6	3.2.4 Disposition of NRC Criteria for Use of LEFM Topical Reports 3.4.3 Plant Modifications 10.5 Operator Training and Human Factors
VII.2.C	The control room plant reference simulator	Attachment 6	10.5 Operator Training and Human Factors
VII.2.D	The operator training program	Attachment 6	10.5 Operator Training and Human Factors
VII.3	Modifications completion	Attachment 1 Attachment 5	3.4.3 Plant Modifications List of Regulatory Commitments
VII.4	Procedure Revisions – Licensed Power Level	Attachment 1	3.2.6 Reactor Power Monitoring

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Item No.	**DESCRIPTION**	**ATTACHMENT**	**SECTION and TITLE**
VII.5.A	10 CFR 51.22, Exclusion of Environmental Review, including discussion of effect of the power uprate on types and amounts of effluents released offsite, and whether bounded by final environmental statement and previous Environmental Assessments for the plant	Attachment 1 Attachment 6	5.0 Environmental Consideration 8 Radwaste and Radiation Sources
VII.5.B	10 CFR 51.22, Exclusion of Environmental Review, including discussion of effect of the power uprate on individual and cumulative occupational radiation exposure	Attachment 1 Attachment 6	5.0 Environmental Consideration 8.5 Radiation Levels
VIII. Changes to Technical Specifications, Protection System Settings, Emergency System Settings			
VIII.1	A detailed discussion of each change to the plant's Technical Specifications, protection system settings, and/or emergency system settings needed to support the power uprate	Attachment 1 Attachment 2	2.0 Detailed Description All Markup of Proposed Operating License and Technical Specifications Pages
VIII.1.A	Description of the change	Attachment 1 Attachment 2	2.0 Detailed Description All Markup of Proposed Operating License and Technical Specifications Pages
VIII.1.B	Identification of analyses affected by and/or supporting the change	Attachment 1 Attachment 6	3.3 Evaluation of Changes to License and Technical Specifications All GEH Nuclear Energy Safety Analysis Report for LaSalle County Station, Units 1 and 2 Thermal Power Optimization, NEDC-33485P

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Item No.	**DESCRIPTION**	**ATTACHMENT**	**SECTION and TITLE**
VIII.1.C	Justification for the change, including the type of information discussed in Section III, above, for any analyses that support and/or are affected by change	Attachment 1	3.3 Evaluation of Changes to License and Technical Specifications
		Attachment 6	All GEH Nuclear Energy Safety Analysis Report for LaSalle County Station, Units 1 and 2 Thermal Power Optimization, NEDC-33485P