

April 7, 2009

LICENSEE: US-APWR DESIGN CENTERED WORKING GROUP

FACILITY: U.S. NUCLEAR REGULATORY COMMISSION

SUBJECT: SUMMARY OF MARCH 9, 2009, PUBLIC MEETING WITH THE UNITED STATES – ADVANCED PRESSURIZED WATER REACTOR DESIGN CENTERED WORKING GROUP

On March 16, 2009, a Category 2 public meeting was held between the U.S. Nuclear Regulatory Commission (NRC) staff and members of the United States – Advanced Pressurized Water Reactor Design-Centered Working Group (DCWG) at the U.S. Nuclear Regulatory Commission (NRC) in Rockville, Maryland. The purpose of this meeting was to discuss Inservice Testing (IST), Inservice Inspection (ISI), and the Reactor Vessel Integrity Programs. The NRC staff had identified a number of issues with both the design certification document (DCD) and combined license (COL) applications. As such, the NRC staff had requested this meeting in order to communicate a successful resolution of these issues to Mitsubishi Heavy Industries (MHI) and Luminant Generation Company. A list of meeting attendees is presented in Enclosure 1. Meeting handouts that were provided to the participants can be found in the Agencywide Documents Access and Management System (ADAMS) under accession numbers ML090760508, ML090760509, and ML090760510.

The NRC staff opened the meeting with a discussion of the IST Program. The NRC staff stated that the DCD needed more detail on the component design process. The COL applicant will need to provide examples of implementing the design process, including procurement specifications. The NRC staff also indicated it was preparing a number of requests for additional information (RAI) that will be sent to MHI. The NRC staff recommended a follow-up conference call with MHI after they had received these RAIs.

Next, the NRC staff discussed the ISI program. The NRC staff presented the attributes of a successful ISI program that would allow the NRC staff to make a reasonable assurance finding. The NRC staff noted that it has already sent RAIs to MHI on the ISI Program and suggested a follow-up conference call to resolve any outstanding questions.

Finally, the NRC staff presented a detailed presentation on the Reactor Vessel Integrity Program. The NRC staff discussed the reactor vessel material surveillance capsule, pressurized thermal shock, upper shelf energy, pressure temperature limits, and fluence protection programs. The DCWG was presented with three options for submitting the pressure temperature limits to the NRC staff. The NRC staff noted that either topical or technical reports are typically used to implement these programs. The DCWG replied they needed time to decide which pressure temperature limits option to utilize.

Members of the public were in attendance, but Public Meeting Feedback forms were not received. Please direct any inquiries to Stephen Monarque at 301-415-1544, or Stephen.Monarque@nrc.gov.

/RA/

Stephen R. Monarque, Project Manager
US-APWR Projects Branch
Division of New Reactor Licensing
Office of New Reactors

Docket Nos. 52-021
52-034 and 52-035

Enclosure:
List of Attendees

cc w/enclos: See next page

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DATE	04/07/2009	04/07/2009

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ATTENDEES

US-APWR DESIGN CENTERED WORKING GROUP PUBLIC MEETING

MARCH 9, 2009

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Jeff Ciocco	NRC/NRO/DNRL
Stephen Raul Monarque	NRC/NRO/DNRL
Jin Chung	MNES
William Ward	NRC/NRO/DNRL
Jeff Poehler	NRC/NRO
Michael Willingham	NRC/NRO
Tim Steingass	NRC/NRO/DE/CIB
Tom Scarbrough	NRC/NRO/DE/CIB
David Terao	NRC/NRO/DE/CIB
Steven Downey	NRC/NRO/DE/CIB
Neil Ray	NRC/NRO/DE/CIB
Carey Bickett	NRC/NRO/DNRL
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Ryan Spengel	MNES
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(Revised 04/01/2009)

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