

Table 1. Table of Contents Proposed Safety Evaluation Review Topics Watts Bar Nuclear Plant Unit 2

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X.Y.Z = chapter.section.subsection.
Chapter designations shown in bold.

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2.5.3	Surface Faulting
2.5.4	Stability of Subsurface Materials and Foundations
2.5.5	Stability of Slopes
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3.1.0	Introduction
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23.2.5	Electrical Issues
23.2.6	Equipment Seismic Qualification
23.2.7	Fire Protection
23.2.8	Hanger and Analysis Update Program
23.2.9	Heat Code Traceability
23.2.10	Heating, Ventilation, and Air-Conditioning Duct and Duct Supports
23.2.11	Instrument Lines
23.2.12	Prestart Test Program
23.2.13	QA Records
23.2.14	Q-List
23.2.15	Replacement Items Program (Piece Parts)
23.2.16	Seismic Analysis
23.2.17	Vendor Information Program

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23.3.1 Concrete Quality Program

23.3.2 Containment Cooling

23.3.3 Detailed Control Room Design Review

23.3.4 Environmental Qualification Program

23.3.5 Master Fuse List

23.3.6 Mechanical Equipment Qualification

23.3.7 Microbiologically Induced Corrosion (MIC)

23.3.8 Moderate Energy Line Break Flooding

23.3.9 Radiation Monitoring System

23.3.10 Soil Liquefaction

23.3.11 Use-as-is CAQs

23.4.0 Implementation, Verification, and Closure of Corrective Actions

23.4.1 Corrective Action Program Plans and Special Programs

23.4.2 Quality Verification Process

23.5.0 Management and Organization

23.5.1 Introduction

23.5.2 Organizational and Management Improvements

23.5.3 Conclusions

23.6.0 Operational Readiness

23.7.0 Employee Concerns

23.8.0 Allegations

24.0.0 Generic Issues

25.0.0 NUREG-0737 TMI Action Items

26.0.0 Other Regulatory Topics

26.1.0 License Conditions

26.2.0 Orders

27.0.0 Conclusions

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Appendix C - Nuclear Regulatory Commission Unresolved Safety Issues

Appendix D - Evaluation of the Applicant's Control Room Design

Appendix E - Principal Contributors

Appendix F - Abbreviations

Table 2.
Status of NUREG-0847 Review Topics
Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>	<u>Resolved in NUREG-0847?*</u>
2.1.1	Site Location and Description	Yes
2.1.2	Exclusion Area Authority and Control	Yes
2.1.3	<i>Population Distribution</i>	No
2.1.4	<i>Conclusions</i>	No
2.2.1	<i>Transportation Routes</i>	No
2.2.2	<i>Nearby Facilities</i>	No
2.2.3	<i>Conclusions</i>	No
2.3.1	Regional Climatology	Yes
2.3.2	Local Meteorology	Yes
2.3.3	Onsite Meteorological Measurements Program	Yes
2.3.4	Short-Term (Accident) Atmospheric Diffusion Estimates	Yes
2.3.5	Long-Term (Routine) Diffusion Estimates	Yes
2.4.1	Introduction	Yes
2.4.2	Hydrologic Description	Yes
2.4.3	Flood Potential	Yes
2.4.4	Local Intense Precipitation in Plant Area	Yes
2.4.5	Roof Drainage	Yes
2.4.6	Ultimate Heat Sink	Yes
2.4.7	Groundwater	Yes
2.4.8	Design Basis for Subsurface Hydrostatic Loading	Yes
2.4.9	<i>Transport of Liquid Releases</i>	No
2.4.10	Flooding Protection Requirements and Technical Specifications	Yes
2.5.0	Geological, Seismological, and Geotechnical Engineering	Yes
2.5.1	Geology	Yes
2.5.2	<i>Seismology</i>	No
2.5.3	Surface Faulting	Yes

*Resolved means that a topic was previously reviewed and approved in NUREG-0847 or its supplements.

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<u>Subsection</u>	<u>Title</u>	<u>Resolved in NUREG-0847?*</u>
2.5.4	<i>Stability of Subsurface Materials and Foundations</i>	No
2.5.5	<i>Stability of Slopes</i>	No
2.5.6	Embankments and Dams	Yes
3.1.1	Conformance With General Design Criteria	Yes
3.1.2	Conformance With Industry Codes and Standards	Yes
3.2.1	Seismic Qualification	Yes
3.2.2	<i>System Quality Group Classification</i>	No
3.3.1	Wind Loading	Yes
3.3.2	Tornado Loading	Yes
3.4.1	Flood Protection	Yes
3.5.1	Missile Selection and Description	Yes
3.5.2	Structures, Systems, and Components To Be Protected From Externally Generated Missiles	Yes
3.5.3	Barrier Design Procedures	Yes
3.6.1	<i>Plant Design for Protection Against Postulated Piping Failures in Fluid</i>	No
3.6.2	Determination of Break Locations and Dynamic Effects Associated with the Postulated Rupture of Piping	Yes
3.6.3	Leak-Before-Break Evaluation Procedures	Yes
3.7.1	<i>Seismic Input</i>	No
3.7.2	<i>Seismic Analysis</i>	No
3.7.3	<i>Seismic Subsystem Analysis</i>	No
3.7.4	Seismic Instrumentation	Yes
3.8.1	Steel Containment	Yes
3.8.2	<i>Concrete and Structural Steel Internal Structures</i>	No
3.8.3	<i>Other Seismic Category I Structures</i>	No
3.8.4	Foundations	Yes
3.9.1	<i>Special Topics for Mechanical Components</i>	No
3.9.2	Dynamic Testing and Analysis of Systems, Components, and Equipment	Yes

*Resolved means that a topic was previously reviewed and approved in NUREG-0847 or its supplements.

Unresolved items are shown in bold italics.

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<u>Subsection</u>	<u>Title</u>	<u>Resolved in NUREG-0847?*</u>
3.9.3	<i>ASME Code Class 1, 2, and 3 Components, Component Structures, and</i>	No
3.9.4	Control Rod Drive Systems	Yes
3.9.5	Reactor Pressure Vessel Internals	Yes
3.9.6	<i>Inservice Testing of Pumps and Valves</i>	No
3.10.0	<i>Seismic and Dynamic Qualification of Seismic Category I Mechanical</i>	No
3.11.0	<i>Environmental Qualification of Mechanical and Electrical Equipment</i>	No
4.2.1	<i>Description</i>	No
4.2.2	<i>Thermal Performance</i>	No
4.2.3	<i>Mechanical Performance</i>	No
4.2.4	Surveillance	Yes
4.2.5	<i>Fuel Design Conclusions</i>	No
4.3.1	<i>Design Basis</i>	No
4.3.2	<i>Design Description</i>	No
4.3.3	<i>Analytical Methods</i>	No
4.3.4	<i>Summary of Evaluation Findings</i>	No
4.4.1	Performance in Safety Criteria	Yes
4.4.2	<i>Design Bases</i>	No
4.4.3	<i>Thermal-Hydraulic Design Methodology</i>	No
4.4.4	<i>Operating Abnormalities</i>	No
4.4.5	<i>Loose Parts Monitoring System</i>	No
4.4.6	Thermal-Hydraulic Comparison	Yes
4.4.7	<i>N-1 Loop Operation</i>	No
4.4.8	<i>Instrumentation for Inadequate Core Cooling Detection (II.F.2)</i>	No
4.4.9	<i>Summary and Conclusion</i>	No
4.5.1	Control Rod Drive Structural Materials	Yes
4.5.2	Reactor Internals and Core Support Materials	Yes

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Unresolved items are shown in bold italics.

Table 2, continued.

Status of NUREG-0847 Review Topics - Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>	<u>Resolved in NUREG-0847?*</u>
4.6.0	Functional Design of Reactivity Control Systems	Yes
5.2.1	Compliance With Codes and Code Cases	Yes
5.2.2	Overpressure Protection	Yes
5.2.3	Reactor Coolant Pressure Boundary Materials	Yes
5.2.4	<i>RCS Pressure Boundary Inservice Inspection and Testing</i>	No
5.2.5	Reactor Coolant Pressure Boundary Leakage Detection	Yes
5.3.1	Reactor Vessel Materials	Yes
5.3.2	<i>Pressure-Temperature Limits</i>	No
5.3.3	<i>Reactor Vessel Integrity</i>	No
5.4.1	Reactor Coolant Pumps	Yes
5.4.2	Steam Generators	Yes
5.4.3	<i>Residual Heat Removal System</i>	No
5.4.4	Pressurizer Relief Tank	Yes
5.4.5	Reactor Coolant System Vents (II.B.1)	Yes
6.1.1	Metallic Materials	Yes
6.1.2	Organic Materials	Yes
6.1.3	Postaccident Emergency Cooling Water Chemistry	Yes
6.2.1	Containment Functional Design	Yes
6.2.2	Containment Heat Removal Systems	Yes
6.2.3	Secondary Containment Functional Design	Yes
6.2.4	Containment Isolation System	Yes
6.2.5	<i>Combustible Gas Control Systems</i>	No
6.2.6	Containment Leakage Testing	Yes
6.2.7	Fracture Prevention of Containment Pressure Boundary	Yes
6.3.0	Emergency Core Cooling System	Yes
6.3.1	<i>System Design</i>	No
6.3.2	<i>Evaluation</i>	No
6.3.3	<i>Testing</i>	No

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Table 2, continued.

Status of NUREG-0847 Review Topics - Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>	<u>Resolved in NUREG-0847?*</u>
6.3.4	Performance Evaluation	Yes
6.3.5	<i>Conclusions</i>	No
6.4.0	Control Room Habitability	Yes
6.5.1	ESF Atmosphere Cleanup Systems	Yes
6.5.2	Fission Product Cleanup System	Yes
6.5.3	Fission Product Control System	Yes
6.5.4	Ice Condenser as a Fission Product Control System	Yes
6.6.0	<i>Inservice Inspection of Class 2 and 3 Components</i>	No
7.1.1	<i>General</i>	No
7.1.2	Comparison with Other Plants	Yes
7.1.3	<i>Design Criteria</i>	No
7.2.0	Reactor Trip System	Yes
7.2.1	<i>System Description</i>	No
7.2.2	Manual Trip Switches	Yes
7.2.3	Testing of Reactor Trip Breaker Shunt Coils	Yes
7.2.4	Anticipatory Trips	Yes
7.2.5	<i>Steam Generator Water Level Trip</i>	No
7.2.6	<i>Conclusions</i>	No
7.3.0	<i>Engineered Safety Features System</i>	No
7.3.1	System Description	Yes
7.3.2	Containment Sump Level Measurement	Yes
7.3.3	Auxiliary Feedwater Initiation and Control	Yes
7.3.4	Failure Modes and Effects Analysis	Yes
7.3.5	IE Bulletin 80-06	Yes
7.3.6	<i>Conclusions</i>	No
7.4.1	System Description	Yes
7.4.2	<i>Safe Shutdown from Auxiliary Control Room</i>	No
7.4.3	Conclusions	Yes

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Table 2, continued.

Status of NUREG-0847 Review Topics - Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>	<u>Resolved in NUREG-0847?*</u>
7.5.1	System Description	Yes
7.5.2	<i>Post-Accident Monitoring System</i>	No
7.5.3	IE Bulletin 79-27	Yes
7.5.4	<i>Conclusions</i>	No
7.6.1	System Description	Yes
7.6.2	Residual Heat Removal System Bypass Valves	Yes
7.6.3	Upper Head Injection Manual Control	Yes
7.6.4	Protection Against Spurious Actuation of Motor-Operated Valves	Yes
7.6.5	Overpressure Protection During Low Temperature Operation	Yes
7.6.6	Valve Power Lockout	Yes
7.6.7	Cold Leg Accumulator Valve Interlocks and Position Indication	Yes
7.6.8	Automatic Switchover From Injection to Recirculation Mode	Yes
7.6.9	Conclusions	Yes
7.7.1	System Description	Yes
7.7.2	<i>Safety System Status Monitoring System</i>	No
7.7.3	Volume Control Tank Level Control System	Yes
7.7.4	Pressurizer and Steam Generator Overfill	Yes
7.7.5	IE Information Notice 79-22	Yes
7.7.6	Multiple Control System Failures	Yes
7.7.7	Conclusions	Yes
7.7.8	<i>Anticipated Transient Without Scram Mitigation System Actuation</i>	No
7.8.0	NUREG-0737 Items	Yes
7.8.1	<i>Relief and Safety Valve Position Indication (II.D.3)</i>	No
7.8.2	Auxiliary Feedwater System Initiation and Flow Indication (II.E.1.2)	Yes
7.8.3	Proportional Integral Derivative Control Modification (II.K.3.9)	Yes
7.8.4	Proposed Anticipatory Trip Modification (II.K.3.10)	Yes
7.8.5	Confirm Existence of Anticipatory Reactor Trip Upon Turbine Trip (II.K.3.12)	Yes

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Table 2, continued.

Status of NUREG-0847 Review Topics - Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>	<u>Resolved in NUREG-0847?*</u>
8.1.0	General	Yes
8.2.1	Compliance With GDC 5	Yes
8.2.2	<i>Compliance With GDC 17</i>	No
8.2.3	Compliance With GDC 18	Yes
8.2.4	Evaluation Findings	Yes
8.3.0	Onsite Power Systems	Yes
8.3.1	<i>Onsite AC Power System Compliance With GDC 17</i>	No
8.3.2	<i>Onsite DC System Compliance With GDC 17</i>	No
8.3.3	<i>Evaluation Findings</i>	No
9.0.0	Auxiliary Systems	Yes
9.1.1	New Fuel Storage	Yes
9.1.2	Spent Fuel Storage	Yes
9.1.3	Spent Fuel Pool Cooling and Cleanup System	Yes
9.1.4	<i>Fuel Handling System</i>	No
9.2.1	<i>Essential Raw Cooling Water and Raw Cooling Water Systems</i>	No
9.2.2	<i>Component Cooling System (Reactor Auxiliaries Cooling Water System)</i>	No
9.2.3	Demineralized Water Makeup System	Yes
9.2.4	Potable and Sanitary Water Systems	Yes
9.2.5	Ultimate Heat Sink	Yes
9.2.6	Condensate Storage Facilities	Yes
9.3.1	Compressed Air System	Yes
9.3.2	<i>Process Sampling System</i>	No
9.3.3	Equipment and Floor Drainage System	Yes
9.3.4	Chemical and Volume Control System	Yes
9.4.1	Control Room Area Ventilation System	Yes
9.4.2	Fuel Handling Area Ventilation System	Yes
9.4.3	Auxiliary and Radwates Area Ventilation System	Yes
9.4.4	Turbine Building Area Ventilation System	Yes

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Table 2, continued.

Status of NUREG-0847 Review Topics - Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>	<u>Resolved in NUREG-0847?*</u>
9.4.5	<i>Engineered Safety Features Ventilation System</i>	No
9.5.1	Fire Protection	Yes
9.5.2	<i>Communication Systems</i>	No
9.5.3	Lighting System	Yes
9.5.4	Emergency Diesel Engine Fuel Oil Storage and Transfer System	Yes
9.5.5	<i>Emergency Diesel Engine Cooling Water System</i>	No
9.5.6	<i>Emergency Diesel Engine Starting Systems</i>	No
9.5.7	<i>Emergency Diesel Engine Lubricating Oil System</i>	No
9.5.8	<i>Emergency Diesel Engine Combustion Air Intake and Exhaust System</i>	No
10.1.0	Summary Description	Yes
10.2.1	Turbine Generator Design	Yes
10.2.2	Turbine Disc Integrity	Yes
10.3.0	Main Steam Supply System	Yes
10.3.1	Main Steam Supply System (up to and including the Main Steam Isolation Valves)	Yes
10.3.2	Main Steam Supply System	Yes
10.3.3	Steam and Feedwater System Materials	Yes
10.3.4	<i>Secondary Water Chemistry</i>	No
10.4.1	Main Condenser	Yes
10.4.2	Main Condenser Evacuation System	Yes
10.4.3	Turbine Gland Sealing System	Yes
10.4.4	Turbine Bypass System	Yes
10.4.5	Condenser Circulating Water System	Yes
10.4.6	Condensate Cleanup System	Yes
10.4.7	Condensate and Feedwater Systems	Yes
10.4.8	Steam Generator Blowdown System	Yes
10.4.9	Auxiliary Feedwater System	Yes
11.1.0	<i>Summary Description</i>	No

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Table 2, continued.

Status of NUREG-0847 Review Topics - Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>	<u>Resolved in NUREG-0847?*</u>
11.2.0	Liquid Waste Management	Yes
11.3.0	Gaseous Waste Management	Yes
11.4.0	<i>Solid Waste Management System</i>	No
11.5.0	<i>Process and Effluent Radiological Monitoring and Sampling Systems</i>	No
11.6.0	<i>Evaluation Findings</i>	No
11.7.0	<i>NUREG-0737 Items</i>	No
11.7.1	<i>Wide Range Noble Gas, Iodine, and Particulate Effluent Monitors</i>	No
11.7.2	<i>Primary Coolant Outside Containment (III.D.1.1)</i>	No
12.1.0	<i>General</i>	No
12.2.0	<i>Ensuring that Occupational Radiation Doses Are As Low As Reasonably</i>	No
12.3.0	<i>Radiation Sources</i>	No
12.4.0	<i>Radiation Protection Design Features</i>	No
12.5.0	<i>Dose Assessment</i>	No
12.6.0	<i>Health Physics Program</i>	No
12.7.1	<i>Plant Shielding (II.B.2)</i>	No
12.7.2	<i>High Range Incontainment Monitor (II.F.1(3))</i>	No
12.7.3	<i>Inplant Radioiodine Monitor (III.D.3.3)</i>	No
13.1.1	Management and Technical Organization	Yes
13.1.2	Corporate Organization and Technical Support	Yes
13.1.3	<i>Plant Staff Organization</i>	No
13.2.1	Licensed Operator Training Program	Yes
13.2.2	Training for Nonlicensed Personnel	Yes
13.3.1	<i>Introduction</i>	No
13.3.2	<i>Evaluation of the Emergency Plan</i>	No
13.3.3	<i>Conclusions</i>	No
13.4.0	<i>Review and Audit</i>	No

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Table 2, continued.

Status of NUREG-0847 Review Topics - Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>	<u>Resolved in NUREG-0847?*</u>
13.5.1	Administrative Procedures	Yes
13.5.2	Operating and Maintenance Procedures	Yes
13.5.3	NUREG-0737 Items	Yes
13.6.0	<i>Physical Security Plan</i>	No
14.0.0	Initial Test Program	Yes
15.1.0	General Discussion	Yes
15.2.0	<i>Normal Operation and Anticipated Transients</i>	No
15.2.1	Loss of Cooling Transients	Yes
15.2.2	Increased Cooling Transients	Yes
15.2.3	<i>Change in Coolant Inventory Transients</i>	No
15.2.4	<i>Reactivity and Power Distribution Anomalies</i>	No
15.2.5	Conclusions	Yes
15.3.0	Limiting Accidents	Yes
15.3.1	<i>Loss-of-Coolant Accident</i>	No
15.3.2	<i>Steamline Break</i>	No
15.3.3	<i>Feedwater System Pipe Break</i>	No
15.3.4	<i>Reactor Coolant Pump Rotor Seizure</i>	No
15.3.5	<i>Reactor Coolant Pump Shaft Break</i>	No
15.3.6	<i>Anticipated Transients Without Scram</i>	No
15.3.7	Conclusions	Yes
15.4.0	Radiological Consequences of Accidents	Yes
15.4.1	<i>Loss-of-Coolant Accident</i>	No
15.4.2	<i>Main Steamline Break Outside of Containment</i>	No
15.4.3	<i>Steam Generator Tube Rupture</i>	No
15.4.4	<i>Control Rod Ejection Accident</i>	No
15.4.5	<i>Fuel-Handling Accident</i>	No
15.4.6	<i>Failure of Small Line Carrying Coolant Outside Containment</i>	No
15.4.7	<i>Postulated Radioactive Releases as a Result of Liquid Tank Failures</i>	No

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Table 2, continued.

Status of NUREG-0847 Review Topics - Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>	<u>Resolved in NUREG-0847?*</u>
15.5.1	<i>Thermal Mechanical Report (II.K.2.13)</i>	No
15.5.2	Voiding in the Reactor Coolant System During Transients (II.K.2.17)	Yes
15.5.3	<i>Installation and Testing of Automatic Power-Operated Relief Valve Isolation System (II.K.3.1) Report on Overall Safety Effect of Power-Operated Relief Valve Isolation</i>	No
15.5.4	<i>Automatic Trip of Reactor Coolant Pumps (II.K.3.5)</i>	No
15.5.5	<i>Small-Break LOCA Methods (II.K.3.30) and Plant-Specific Calculations</i>	No
15.6.0	Relative Risk of Low Power Operation	Yes
16.0.0	<i>Technical Specifications</i>	No
17.1.0	General	Yes
17.2.0	Organization	Yes
17.3.0	Quality Assurance Program	Yes
17.4.0	Conclusions	Yes
18.1.0	<i>General</i>	No
18.2.0	<i>Conclusions</i>	No
20.0.0	Common Defense and Security	Yes
21.0.0	Financial Qualifications	Yes
22.1.0	General	Yes
22.2.0	Preoperational Storage of Nuclear Fuel	Yes
22.3.0	Operating Licenses	Yes

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Table 3.
Review Topics Resolved in NUREG-0847
Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
2.1.1	Site Location and Description
2.1.2	Exclusion Area Authority and Control
2.3.1	Regional Climatology
2.3.2	Local Meteorology
2.3.3	Onsite Meteorological Measurements Program
2.3.4	Short-Term (Accident) Atmospheric Diffusion Estimates
2.3.5	Long-Term (Routine) Diffusion Estimates
2.4.1	Introduction
2.4.2	Hydrologic Description
2.4.3	Flood Potential
2.4.4	Local Intense Precipitation in Plant Area
2.4.5	Roof Drainage
2.4.6	Ultimate Heat Sink
2.4.7	Groundwater
2.4.8	Design Basis for Subsurface Hydrostatic Loading
2.4.10	Flooding Protection Requirements and Technical Specifications
2.5.0	Geological, Seismological, and Geotechnical Engineering
2.5.1	Geology
2.5.3	Surface Faulting
2.5.6	Embankments and Dams
3.1.1	Conformance With General Design Criteria
3.1.2	Conformance With Industry Codes and Standards
3.2.1	Seismic Qualification
3.3.1	Wind Loading
3.3.2	Tornado Loading
3.4.1	Flood Protection

Table 3, continued.

Review Topics Resolved in NUREG-0847 - Watts Bar Nuclear Plant Unit 2

Subsection Title

- 3.5.1 Missile Selection and Description
- 3.5.2 Structures, Systems, and Components To Be Protected From Externally Generated Missiles
- 3.5.3 Barrier Design Procedures
- 3.6.2 Determination of Break Locations and Dynamic Effects Associated with the Postulated Rupture of Piping
- 3.6.3 Leak-Before-Break Evaluation Procedures
- 3.7.4 Seismic Instrumentation
- 3.8.1 Steel Containment
- 3.8.4 Foundations
- 3.9.2 Dynamic Testing and Analysis of Systems, Components, and Equipment
- 3.9.4 Control Rod Drive Systems
- 3.9.5 Reactor Pressure Vessel Internals
- 4.2.4 Surveillance
- 4.4.1 Performance in Safety Criteria
- 4.4.6 Thermal-Hydraulic Comparison
- 4.5.1 Control Rod Drive Structural Materials
- 4.5.2 Reactor Internals and Core Support Materials
- 4.6.0 Functional Design of Reactivity Control Systems
- 5.2.1 Compliance With Codes and Code Cases
- 5.2.2 Overpressure Protection
- 5.2.3 Reactor Coolant Pressure Boundary Materials
- 5.2.5 Reactor Coolant Pressure Boundary Leakage Detection
- 5.3.1 Reactor Vessel Materials
- 5.4.1 Reactor Coolant Pumps
- 5.4.2 Steam Generators
- 5.4.4 Pressurizer Relief Tank
- 5.4.5 Reactor Coolant System Vents (II.B.1)

Table 3, continued.

Review Topics Resolved in NUREG-0847 - Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
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- | | |
|-------|--|
| 6.1.1 | Metallic Materials |
| 6.1.2 | Organic Materials |
| 6.1.3 | Postaccident Emergency Cooling Water Chemistry |
| 6.2.1 | Containment Functional Design |
| 6.2.2 | Containment Heat Removal Systems |
| 6.2.3 | Secondary Containment Functional Design |
| 6.2.4 | Containment Isolation System |
| 6.2.6 | Containment Leakage Testing |
| 6.2.7 | Fracture Prevention of Containment Pressure Boundary |
| 6.3.0 | Emergency Core Cooling System |
| 6.3.4 | Performance Evaluation |
| 6.4.0 | Control Room Habitability |
| 6.5.1 | ESF Atmosphere Cleanup Systems |
| 6.5.2 | Fission Product Cleanup System |
| 6.5.3 | Fission Product Control System |
| 6.5.4 | Ice Condenser as a Fission Product Control System |
| 7.1.2 | Comparison with Other Plants |
| 7.2.0 | Reactor Trip System |
| 7.2.2 | Manual Trip Switches |
| 7.2.3 | Testing of Reactor Trip Breaker Shunt Coils |
| 7.2.4 | Anticipatory Trips |
| 7.3.1 | System Description |
| 7.3.2 | Containment Sump Level Measurement |
| 7.3.3 | Auxiliary Feedwater Initiation and Control |
| 7.3.4 | Failure Modes and Effects Analysis |
| 7.3.5 | IE Bulletin 80-06 |
| 7.4.1 | System Description |

Table 3, continued.

Review Topics Resolved in NUREG-0847 - Watts Bar Nuclear Plant Unit 2

Subsection Title

- 7.4.3 Conclusions
- 7.5.1 System Description
- 7.5.3 IE Bulletin 79-27
- 7.6.1 System Description
- 7.6.2 Residual Heat Removal System Bypass Valves
- 7.6.3 Upper Head Injection Manual Control
- 7.6.4 Protection Against Spurious Actuation of Motor-Operated Valves
- 7.6.5 Overpressure Protection During Low Temperature Operation
- 7.6.6 Valve Power Lockout
- 7.6.7 Cold Leg Accumulator Valve Interlocks and Position Indication
- 7.6.8 Automatic Switchover From Injection to Recirculation Mode
- 7.6.9 Conclusions
- 7.7.1 System Description
- 7.7.3 Volume Control Tank Level Control System
- 7.7.4 Pressurizer and Steam Generator Overfill
- 7.7.5 IE Information Notice 79-22
- 7.7.6 Multiple Control System Failures
- 7.7.7 Conclusions
- 7.8.0 NUREG-0737 Items
- 7.8.2 Auxiliary Feedwater System Initiation and Flow Indication (II.E.1.2)
- 7.8.3 Proportional Integral Derivative Control Modification (II.K.3.9)
- 7.8.4 Proposed Anticipatory Trip Modification (II.K.3.10)
- 7.8.5 Confirm Existence of Anticipatory Reactor Trip Upon Turbine Trip (II.K.3.12)
- 8.1.0 General
- 8.2.1 Compliance With GDC 5
- 8.2.3 Compliance With GDC 18
- 8.2.4 Evaluation Findings

Table 3, continued.

Review Topics Resolved in NUREG-0847 - Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
8.3.0	Onsite Power Systems
9.0.0	Auxiliary Systems
9.1.1	New Fuel Storage
9.1.2	Spent Fuel Storage
9.1.3	Spent Fuel Pool Cooling and Cleanup System
9.2.3	Demineralized Water Makeup System
9.2.4	Potable and Sanitary Water Systems
9.2.5	Ultimate Heat Sink
9.2.6	Condensate Storage Facilities
9.3.1	Compressed Air System
9.3.3	Equipment and Floor Drainage System
9.3.4	Chemical and Volume Control System
9.4.1	Control Room Area Ventilation System
9.4.2	Fuel Handling Area Ventilation System
9.4.3	Auxiliary and Radwates Area Ventilation System
9.4.4	Turbine Building Area Ventilation System
9.5.1	Fire Protection
9.5.3	Lighting System
9.5.4	Emergency Diesel Engine Fuel Oil Storage and Transfer System
10.1.0	Summary Description
10.2.1	Turbine Generator Design
10.2.2	Turbine Disc Integrity
10.3.0	Main Steam Supply System
10.3.1	Main Steam Supply System (up to and including the Main Steam Isolation Valves)
10.3.2	Main Steam Supply System
10.3.3	Steam and Feedwater System Materials

Table 3, continued.

Review Topics Resolved in NUREG-0847 - Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
10.4.1	Main Condenser
10.4.2	Main Condenser Evacuation System
10.4.3	Turbine Gland Sealing System
10.4.4	Turbine Bypass System
10.4.5	Condenser Circulating Water System
10.4.6	Condensate Cleanup System
10.4.7	Condensate and Feedwater Systems
10.4.8	Steam Generator Blowdown System
10.4.9	Auxiliary Feedwater System
11.2.0	Liquid Waste Management
11.3.0	Gaseous Waste Management
13.1.1	Management and Technical Organization
13.1.2	Corporate Organization and Technical Support
13.2.1	Licensed Operator Training Program
13.2.2	Training for Nonlicensed Personnel
13.5.1	Administrative Procedures
13.5.2	Operating and Maintenance Procedures
13.5.3	NUREG-0737 Items
14.0.0	Initial Test Program
15.1.0	General Discussion
15.2.1	Loss of Cooling Transients
15.2.2	Increased Cooling Transients
15.2.5	Conclusions
15.3.0	Limiting Accidents
15.3.7	Conclusions
15.4.0	Radiological Consequences of Accidents
15.5.2	Voiding in the Reactor Coolant System During Transients (II.K.2.17)

Table 3, continued.

Review Topics Resolved in NUREG-0847 - Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
15.6.0	Relative Risk of Low Power Operation
17.1.0	General
17.2.0	Organization
17.3.0	Quality Assurance Program
17.4.0	Conclusions
20.0.0	Common Defense and Security
21.0.0	Financial Qualifications
22.1.0	General
22.2.0	Preoperational Storage of Nuclear Fuel
22.3.0	Operating Licenses

Table 4.
Review Topics Unresolved in NUREG-0847
Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
2.1.3	Population Distribution
2.1.4	Conclusions
2.2.1	Transportation Routes
2.2.2	Nearby Facilities
2.2.3	Conclusions
2.4.9	Transport of Liquid Releases
2.5.2	Seismology
2.5.4	Stability of Subsurface Materials and Foundations
2.5.5	Stability of Slopes
3.2.2	System Quality Group Classification
3.6.1	Plant Design for Protection Against Postulated Piping Failures in Fluid Systems Outside Containment
3.7.1	Seismic Input
3.7.2	Seismic Analysis
3.7.3	Seismic Subsystem Analysis
3.8.2	Concrete and Structural Steel Internal Structures
3.8.3	Other Seismic Category I Structures
3.9.1	Special Topics for Mechanical Components
3.9.3	ASME Code Class 1, 2, and 3 Components, Component Structures, and Core Support Structures
3.9.6	Inservice Testing of Pumps and Valves
3.10.0	Seismic and Dynamic Qualification of Seismic Category I Mechanical and Electrical Equipment
3.11.0	Environmental Qualification of Mechanical and Electrical Equipment
4.2.1	Description
4.2.2	Thermal Performance
4.2.3	Mechanical Performance

Table 4, continued.

Review Topics Unresolved in NUREG-0847 - Watts Bar Nuclear Plant Unit 2

Subsection Title

- 4.2.5 Fuel Design Conclusions
 - 4.3.1 Design Basis
 - 4.3.2 Design Description
 - 4.3.3 Analytical Methods
 - 4.3.4 Summary of Evaluation Findings
- 4.4.2 Design Bases
 - 4.4.3 Thermal-Hydraulic Design Methodology
 - 4.4.4 Operating Abnormalities
 - 4.4.5 Loose Parts Monitoring System
 - 4.4.7 N-1 Loop Operation
 - 4.4.8 Instrumentation for Inadequate Core Cooling Detection (II.F.2)
 - 4.4.9 Summary and Conclusion
- 5.2.4 RCS Pressure Boundary Inservice Inspection and Testing
- 5.3.2 Pressure-Temperature Limits
 - 5.3.3 Reactor Vessel Integrity
- 5.4.3 Residual Heat Removal System
- 6.2.5 Combustible Gas Control Systems
 - 6.3.1 System Design
 - 6.3.2 Evaluation
 - 6.3.3 Testing
 - 6.3.5 Conclusions
- 6.6.0 Inservice Inspection of Class 2 and 3 Components
 - 7.1.1 General
 - 7.1.3 Design Criteria
 - 7.2.1 System Description
 - 7.2.5 Steam Generator Water Level Trip
 - 7.2.6 Conclusions
 - 7.3.0 Engineered Safety Features System

Table 4, continued.

Review Topics Unresolved in NUREG-0847 - Watts Bar Nuclear Plant Unit 2

Subsection Title

- 7.3.6 Conclusions
- 7.4.2 Safe Shutdown from Auxiliary Control Room
- 7.5.2 Post-Accident Monitoring System
- 7.5.4 Conclusions
- 7.7.2 Safety System Status Monitoring System
- 7.7.8 Anticipated Transient Without Scram Mitigation System Actuation Circuitry (AMSAC)
- 7.8.1 Relief and Safety Valve Position Indication (II.D.3)
- 8.2.2 Compliance With GDC 17
- 8.3.1 Onsite AC Power System Compliance With GDC 17
- 8.3.2 Onsite DC System Compliance With GDC 17
- 8.3.3 Evaluation Findings
- 9.1.4 Fuel Handling System
- 9.2.1 Essential Raw Cooling Water and Raw Cooling Water Systems
- 9.2.2 Component Cooling System (Reactor Auxiliaries Cooling Water System)
- 9.3.2 Process Sampling System
- 9.4.5 Engineered Safety Features Ventilation System
- 9.5.2 Communication Systems
- 9.5.5 Emergency Diesel Engine Cooling Water System
- 9.5.6 Emergency Diesel Engine Starting Systems
- 9.5.7 Emergency Diesel Engine Lubricating Oil System
- 9.5.8 Emergency Diesel Engine Combustion Air Intake and Exhaust System
- 10.3.4 Secondary Water Chemistry
- 11.1.0 Summary Description
- 11.4.0 Solid Waste Management System
- 11.5.0 Process and Effluent Radiological Monitoring and Sampling Systems
- 11.6.0 Evaluation Findings
- 11.7.0 NUREG-0737 Items

Table 4, continued.

Review Topics Unresolved in NUREG-0847 - Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
11.7.1	Wide Range Noble Gas, Iodine, and Particulate Effluent Monitors (II.F.1(1) and II.F.1(2))
11.7.2	Primary Coolant Outside Containment (III.D.1.1)
12.1.0	General
12.2.0	Ensuring that Occupational Radiation Doses Are As Low As Reasonably Achievable
12.3.0	Radiation Sources
12.4.0	Radiation Protection Design Features
12.5.0	Dose Assessment
12.6.0	Health Physics Program
12.7.1	Plant Shielding (II.B.2)
12.7.2	High Range Incontainment Monitor (II.F.1(3))
12.7.3	Inplant Radioiodine Monitor (III.D.3.3)
13.1.3	Plant Staff Organization
13.3.1	Introduction
13.3.2	Evaluation of the Emergency Plan
13.3.3	Conclusions
13.4.0	Review and Audit
13.6.0	Physical Security Plan
15.2.0	Normal Operation and Anticipated Transients
15.2.3	Change in Coolant Inventory Transients
15.2.4	Reactivity and Power Distribution Anomalies
15.3.1	Loss-of-Coolant Accident
15.3.2	Steamline Break
15.3.3	Feedwater System Pipe Break
15.3.4	Reactor Coolant Pump Rotor Seizure
15.3.5	Reactor Coolant Pump Shaft Break
15.3.6	Anticipated Transients Without Scram

Table 4, continued.

Review Topics Unresolved in NUREG-0847 - Watts Bar Nuclear Plant Unit 2

Subsection Title

- 15.4.1 Loss-of-Coolant Accident
- 15.4.2 Main Steamline Break Outside of Containment
- 15.4.3 Steam Generator Tube Rupture
- 15.4.4 Control Rod Ejection Accident
- 15.4.5 Fuel-Handling Accident
- 15.4.6 Failure of Small Line Carrying Coolant Outside Containment
- 15.4.7 Postulated Radioactive Releases as a Result of Liquid Tank Failures
- 15.5.1 Thermal Mechanical Report (II.K.2.13)
- 15.5.3 Installation and Testing of Automatic Power-Operated Relief Valve Isolation System (II.K.3.1)
Report on Overall Safety Effect of Power-Operated Relief Valve Isolation System (II.K.3.2)
- 15.5.4 Automatic Trip of Reactor Coolant Pumps (II.K.3.5)
- 15.5.5 Small-Break LOCA Methods (II.K.3.30) and Plant-Specific Calculations (II.K.3.31)
- 16.0.0 Technical Specifications
- 18.1.0 General
- 18.2.0 Conclusions

Table 5.
NUREG-0847 Review Topics Where NRC Conclusions Do Not
Match TVA
(NRC = Open, TVA = Closed)

<u>Subsection</u>	<u>Title</u>
2.5.2	Seismology
2.5.4	Stability of Subsurface Materials and Foundations
2.5.5	Stability of Slopes
3.2.2	System Quality Group Classification
3.6.1	Plant Design for Protection Against Postulated Piping Failures in Fluid Systems Outside Containment
3.7.1	Seismic Input
3.7.2	Seismic Analysis
3.8.2	Concrete and Structural Steel Internal Structures
3.8.3	Other Seismic Category I Structures
3.9.1	Special Topics for Mechanical Components
3.9.3	ASME Code Class 1, 2, and 3 Components, Component Structures, and Core Support Structures
3.9.6	Inservice Testing of Pumps and Valves
3.10.0	Seismic and Dynamic Qualification of Seismic Category I Mechanical and Electrical Equipment
4.2.1	Description
4.2.3	Mechanical Performance
4.3.1	Design Basis
4.3.2	Design Description
4.3.3	Analytical Methods
4.3.4	Summary of Evaluation Findings
4.4.2	Design Bases
4.4.4	Operating Abnormalities
4.4.7	N-1 Loop Operation
5.4.3	Residual Heat Removal System
6.3.1	System Design

Table 5, continued.

NUREG-0847 Review Topics Where NRC Conclusions Do Not Match TVA
(NRC = Open, TVA = Closed)

<u>Subsection</u>	<u>Title</u>
6.3.2	Evaluation
6.3.3	Testing
6.3.5	Conclusions
7.1.1	General
7.2.1	System Description
7.2.5	Steam Generator Water Level Trip
7.3.0	Engineered Safety Features System
7.4.2	Safe Shutdown from Auxiliary Control Room
7.5.2	Post-Accident Monitoring System
7.7.2	Safety System Status Monitoring System
7.7.8	Anticipated Transient Without Scram Mitigation System Actuation Circuitry (AMSAC)
7.8.1	Relief and Safety Valve Position Indication (II.D.3)
8.2.2	Compliance With GDC 17
8.3.1	Onsite AC Power System Compliance With GDC 17
8.3.2	Onsite DC System Compliance With GDC 17
8.3.3	Evaluation Findings
9.1.4	Fuel Handling System
9.2.2	Component Cooling System (Reactor Auxiliaries Cooling Water System)
9.3.2	Process Sampling System
9.4.5	Engineered Safety Features Ventilation System
9.5.2	Communication Systems
9.5.5	Emergency Diesel Engine Cooling Water System
9.5.6	Emergency Diesel Engine Starting Systems
9.5.7	Emergency Diesel Engine Lubricating Oil System
9.5.8	Emergency Diesel Engine Combustion Air Intake and Exhaust System
11.1.0	Summary Description

Table 5, continued.

NUREG-0847 Review Topics Where NRC Conclusions Do Not Match TVA
(NRC = Open, TVA = Closed)

<u>Subsection</u>	<u>Title</u>
11.6.0	Evaluation Findings
11.7.0	NUREG-0737 Items
11.7.1	Wide Range Noble Gas, Iodine, and Particulate Effluent Monitors (II.F.1(1) and II.F.1(2))
12.1.0	General
12.2.0	Ensuring that Occupational Radiation Doses Are As Low As Reasonably Achievable
12.3.0	Radiation Sources
12.4.0	Radiation Protection Design Features
12.5.0	Dose Assessment
12.6.0	Health Physics Program
12.7.1	Plant Shielding (II.B.2)
12.7.2	High Range Incontainment Monitor (II.F.1(3))
12.7.3	Inplant Radioiodine Monitor (III.D.3.3)
13.1.3	Plant Staff Organization
13.3.1	Introduction
13.3.2	Evaluation of the Emergency Plan
13.3.3	Conclusions
13.6.0	Physical Security Plan
15.2.0	Normal Operation and Anticipated Transients
15.2.3	Change in Coolant Inventory Transients
15.2.4	Reactivity and Power Distribution Anomalies
15.3.1	Loss-of-Coolant Accident
15.3.2	Steamline Break
15.3.3	Feedwater System Pipe Break
15.3.4	Reactor Coolant Pump Rotor Seizure
15.3.5	Reactor Coolant Pump Shaft Break
15.3.6	Anticipated Transients Without Scram

Table 5, continued.

NUREG-0847 Review Topics Where NRC Conclusions Do Not Match TVA
(NRC = Open, TVA = Closed)

Subsection Title

- 15.4.1 Loss-of-Coolant Accident
- 15.4.2 Main Steamline Break Outside of Containment
- 15.4.3 Steam Generator Tube Rupture
- 15.4.4 Control Rod Ejection Accident
- 15.4.5 Fuel-Handling Accident
- 15.4.6 Failure of Small Line Carrying Coolant Outside Containment
- 15.4.7 Postulated Radioactive Releases as a Result of Liquid Tank Failures
- 15.5.1 Thermal Mechanical Report (II.K.2.13)
- 15.5.3 Installation and Testing of Automatic Power-Operated Relief Valve Isolation System (II.K.3.1)
Report on Overall Safety Effect of Power-Operated Relief Valve Isolation System (II.K.3.2)
- 15.5.4 Automatic Trip of Reactor Coolant Pumps (II.K.3.5)
- 15.5.5 Small-Break LOCA Methods (II.K.3.30) and Plant-Specific Calculations (II.K.3.31)

Table 6.
Open NUREG-0847 Review Topics Requiring Additional
Information to Resolve

<u>Subsection</u>	<u>Title</u>
2.5.2	Seismology
2.5.4	Stability of Subsurface Materials and Foundations
2.5.5	Stability of Slopes
3.2.2	System Quality Group Classification
3.6.1	Plant Design for Protection Against Postulated Piping Failures in Fluid Systems Outside Containment
3.7.1	Seismic Input
3.7.2	Seismic Analysis
3.8.2	Concrete and Structural Steel Internal Structures
3.8.3	Other Seismic Category I Structures
3.9.1	Special Topics for Mechanical Components
3.9.3	ASME Code Class 1, 2, and 3 Components, Component Structures, and Core Support Structures
3.10.0	Seismic and Dynamic Qualification of Seismic Category I Mechanical and Electrical Equipment
4.2.1	Description
4.2.3	Mechanical Performance
4.3.1	Design Basis
4.3.2	Design Description
4.3.3	Analytical Methods
4.3.4	Summary of Evaluation Findings
4.4.2	Design Bases
4.4.4	Operating Abnormalities
5.4.3	Residual Heat Removal System
6.3.1	System Design
6.3.2	Evaluation
6.3.3	Testing

Table 6, continued.

Open NUREG-0847 Review Topics Requiring Additional Information to Resolve

<u>Subsection</u>	<u>Title</u>
6.3.5	Conclusions
7.1.1	General
7.2.1	System Description
7.2.5	Steam Generator Water Level Trip
7.3.0	Engineered Safety Features System
7.4.2	Safe Shutdown from Auxiliary Control Room
7.5.2	Post-Accident Monitoring System
7.7.2	Safety System Status Monitoring System
7.8.1	Relief and Safety Valve Position Indication (II.D.3)
8.2.2	Compliance With GDC 17
8.3.1	Onsite AC Power System Compliance With GDC 17
8.3.2	Onsite DC System Compliance With GDC 17
8.3.3	Evaluation Findings
9.1.4	Fuel Handling System
9.2.2	Component Cooling System (Reactor Auxiliaries Cooling Water System)
9.3.2	Process Sampling System
9.4.5	Engineered Safety Features Ventilation System
9.5.2	Communication Systems
9.5.5	Emergency Diesel Engine Cooling Water System
9.5.6	Emergency Diesel Engine Starting Systems
9.5.7	Emergency Diesel Engine Lubricating Oil System
9.5.8	Emergency Diesel Engine Combustion Air Intake and Exhaust System
11.1.0	Summary Description
11.6.0	Evaluation Findings
11.7.0	NUREG-0737 Items
11.7.1	Wide Range Noble Gas, Iodine, and Particulate Effluent Monitors (II.F.1(1) and II.F.1(2))
12.1.0	General

Table 6, continued.

Open NUREG-0847 Review Topics Requiring Additional Information to Resolve

<u>Subsection</u>	<u>Title</u>
12.2.0	Ensuring that Occupational Radiation Doses Are As Low As Reasonably Achievable
12.3.0	Radiation Sources
12.4.0	Radiation Protection Design Features
12.5.0	Dose Assessment
12.6.0	Health Physics Program
12.7.1	Plant Shielding (II.B.2)
12.7.2	High Range Incontainment Monitor (II.F.1(3))
12.7.3	Inplant Radioiodine Monitor (III.D.3.3)
13.1.3	Plant Staff Organization
13.3.1	Introduction
13.3.2	Evaluation of the Emergency Plan
13.3.3	Conclusions
13.6.0	Physical Security Plan
15.2.0	Normal Operation and Anticipated Transients
15.2.3	Change in Coolant Inventory Transients
15.2.4	Reactivity and Power Distribution Anomalies
15.3.1	Loss-of-Coolant Accident
15.3.2	Steamline Break
15.3.3	Feedwater System Pipe Break
15.3.4	Reactor Coolant Pump Rotor Seizure
15.3.5	Reactor Coolant Pump Shaft Break
15.3.6	Anticipated Transients Without Scram
15.4.1	Loss-of-Coolant Accident
15.4.2	Main Steamline Break Outside of Containment
15.4.3	Steam Generator Tube Rupture
15.4.4	Control Rod Ejection Accident
15.4.5	Fuel-Handling Accident

Table 6, continued.

Open NUREG-0847 Review Topics Requiring Additional Information to Resolve

Subsection Title

- 15.4.6 Failure of Small Line Carrying Coolant Outside Containment
- 15.4.7 Postulated Radioactive Releases as a Result of Liquid Tank Failures
- 15.5.1 Thermal Mechanical Report (II.K.2.13)
- 15.5.3 Installation and Testing of Automatic Power-Operated Relief Valve Isolation System (II.K.3.1)
Report on Overall Safety Effect of Power-Operated Relief Valve Isolation System (II.K.3.2)
- 15.5.4 Automatic Trip of Reactor Coolant Pumps (II.K.3.5)
- 15.5.5 Small-Break LOCA Methods (II.K.3.30) and Plant-Specific Calculations (II.K.3.31)

Table 7.
NUREG-0847 and Supplemental Safety Evaluations Topic Inventory
Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
<u>2.1.1</u>	<u>Site Location and Description</u> NUREG-0847 Supplement 0
<u>2.1.2</u>	<u>Exclusion Area Authority and Control</u> NUREG-0847 Supplement 0
<u>2.1.3</u>	<u>Population Distribution</u> NUREG-0847 Supplement 0
<u>2.1.4</u>	<u>Conclusions</u> NUREG-0847 Supplement 0
<u>2.2.1</u>	<u>Transportation Routes</u> NUREG-0847 Supplement 0
<u>2.2.2</u>	<u>Nearby Facilities</u> NUREG-0847 Supplement 0
<u>2.2.3</u>	<u>Conclusions</u> NUREG-0847 Supplement 0
<u>2.3.1</u>	<u>Regional Climatology</u> NUREG-0847 Supplement 0
<u>2.3.2</u>	<u>Local Meteorology</u> NUREG-0847 Supplement 0
<u>2.3.3</u>	<u>Onsite Meteorological Measurements Program</u> NUREG-0847 Supplement 0
<u>2.3.4</u>	<u>Short-Term (Accident) Atmospheric Diffusion Estimates</u> NUREG-0847 Supplement 0
<u>2.3.5</u>	<u>Long-Term (Routine) Diffusion Estimates</u> NUREG-0847 Supplement 0
<u>2.4.1</u>	<u>Introduction</u> NUREG-0847 Supplement 0
<u>2.4.2</u>	<u>Hydrologic Description</u> NUREG-0847 Supplement 0

Table 7.
 NUREG-0847 and Supplemental Safety Evaluations Topic Inventory
 Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
<u>2.4.3</u>	<u>Flood Potential</u> NUREG-0847 Supplement 0
<u>2.4.4</u>	<u>Local Intense Precipitation in Plant Area</u> NUREG-0847 Supplement 0
<u>2.4.5</u>	<u>Roof Drainage</u> NUREG-0847 Supplement 0
<u>2.4.6</u>	<u>Ultimate Heat Sink</u> NUREG-0847 Supplement 0
<u>2.4.7</u>	<u>Groundwater</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 3
<u>2.4.8</u>	<u>Design Basis for Subsurface Hydrostatic Loading</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 3
<u>2.4.9</u>	<u>Transport of Liquid Releases</u> NUREG-0847 Supplement 0
<u>2.4.10</u>	<u>Flooding Protection Requirements and Technical Specifications</u> NUREG-0847 Supplement 0
<u>2.5.0</u>	<u>Geological, Seismological, and Geotechnical Engineering</u> NUREG-0847 Supplement 0
<u>2.5.1</u>	<u>Geology</u> NUREG-0847 Supplement 0
<u>2.5.2</u>	<u>Seismology</u> NUREG-0847 Supplement 0
<u>2.5.3</u>	<u>Surface Faulting</u> NUREG-0847 Supplement 0

Table 7.
NUREG-0847 and Supplemental Safety Evaluations Topic Inventory
Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
<u>2.5.4</u>	<u>Stability of Subsurface Materials and Foundations</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 3 NUREG-0847 Supplement 9 NUREG-0847 Supplement 11
<u>2.5.5</u>	<u>Stability of Slopes</u> NUREG-0847 Supplement 0
<u>2.5.6</u>	<u>Embankments and Dams</u> NUREG-0847 Supplement 0
<u>3.1.1</u>	<u>Conformance With General Design Criteria</u> NUREG-0847 Supplement 0
<u>3.1.2</u>	<u>Conformance With Industry Codes and Standards</u> NUREG-0847 Supplement 0
<u>3.2.1</u>	<u>Seismic Qualification</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 3 NUREG-0847 Supplement 5 NUREG-0847 Supplement 6 NUREG-0847 Supplement 8
<u>3.2.2</u>	<u>System Quality Group Classification</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 3 NUREG-0847 Supplement 6 NUREG-0847 Supplement 7 NUREG-0847 Supplement 9
<u>3.3.1</u>	<u>Wind Loading</u> NUREG-0847 Supplement 0
<u>3.3.2</u>	<u>Tornado Loading</u> NUREG-0847 Supplement 0
<u>3.4.1</u>	<u>Flood Protection</u> NUREG-0847 Supplement 0

Table 7.
 NUREG-0847 and Supplemental Safety Evaluations Topic Inventory
 Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
<u>3.5.1</u>	<u>Missile Selection and Description</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 9 NUREG-0847 Supplement 14
<u>3.5.2</u>	<u>Structures, Systems, and Components To Be Protected From Externally Generated Missiles</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 2
<u>3.5.3</u>	<u>Barrier Design Procedures</u> NUREG-0847 Supplement 0
<u>3.6.1</u>	<u>Plant Design for Protection Against Postulated Piping Failures in Fluid Systems Outside Containment</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 14
<u>3.6.2</u>	<u>Determination of Break Locations and Dynamic Effects Associated with the Postulated Rupture of Piping</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 14
<u>3.6.3</u>	<u>Leak-Before-Break Evaluation Procedures</u> NUREG-0847 Supplement 5
<u>3.7.1</u>	<u>Seismic Input</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 6 NUREG-0847 Supplement 9 NUREG-0847 Supplement 16
<u>3.7.2</u>	<u>Seismic Analysis</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 6 NUREG-0847 Supplement 8 NUREG-0847 Supplement 11 NUREG-0847 Supplement 16

Table 7.
NUREG-0847 and Supplemental Safety Evaluations Topic Inventory
Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
<u>3.7.3</u>	<u>Seismic Subsystem Analysis</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 6
	NUREG-0847 Supplement 7
	NUREG-0847 Supplement 8
	NUREG-0847 Supplement 9
	NUREG-0847 Supplement 12
<u>3.7.4</u>	<u>Seismic Instrumentation</u>
	NUREG-0847 Supplement 0
<u>3.8.1</u>	<u>Steel Containment</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 3
<u>3.8.2</u>	<u>Concrete and Structural Steel Internal Structures</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 7
<u>3.8.3</u>	<u>Other Seismic Category I Structures</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 14
	NUREG-0847 Supplement 16
<u>3.8.4</u>	<u>Foundations</u>
	NUREG-0847 Supplement 0
<u>3.9.1</u>	<u>Special Topics for Mechanical Components</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 6
	NUREG-0847 Supplement 13
<u>3.9.2</u>	<u>Dynamic Testing and Analysis of Systems, Components, and Equipment</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 14

Table 7.
 NUREG-0847 and Supplemental Safety Evaluations Topic Inventory
 Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
<u>3.9.3</u>	<u>ASME Code Class 1, 2, and 3 Components, Component Structures, and Core Support Structures</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 3
	NUREG-0847 Supplement 4
	NUREG-0847 Supplement 6
	NUREG-0847 Supplement 7
	NUREG-0847 Supplement 8
	NUREG-0847 Supplement 15
<u>3.9.4</u>	<u>Control Rod Drive Systems</u>
	NUREG-0847 Supplement 0
<u>3.9.5</u>	<u>Reactor Pressure Vessel Internals</u>
	NUREG-0847 Supplement 0
<u>3.9.6</u>	<u>Inservice Testing of Pumps and Valves</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 5
	NUREG-0847 Supplement 12
	NUREG-0847 Supplement 14
	NUREG-0847 Supplement 20
<u>3.10.0</u>	<u>Seismic and Dynamic Qualification of Seismic Category I Mechanical and Electrical Equipment</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 1
	NUREG-0847 Supplement 3
	NUREG-0847 Supplement 4
	NUREG-0847 Supplement 5
	NUREG-0847 Supplement 6
	NUREG-0847 Supplement 8
	NUREG-0847 Supplement 9
<u>3.11.0</u>	<u>Environmental Qualification of Mechanical and Electrical Equipment</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 15

Table 7.
NUREG-0847 and Supplemental Safety Evaluations Topic Inventory
Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
<u>4.2.1</u>	<u>Description</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 13
<u>4.2.2</u>	<u>Thermal Performance</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 2
<u>4.2.3</u>	<u>Mechanical Performance</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 2
	NUREG-0847 Supplement 10
	NUREG-0847 Supplement 13
<u>4.2.4</u>	<u>Surveillance</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 2
<u>4.2.5</u>	<u>Fuel Design Conclusions</u>
	NUREG-0847 Supplement 0
<u>4.3.1</u>	<u>Design Basis</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 13
<u>4.3.2</u>	<u>Design Description</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 13
	NUREG-0847 Supplement 15
<u>4.3.3</u>	<u>Analytical Methods</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 13
<u>4.3.4</u>	<u>Summary of Evaluation Findings</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 13
<u>4.4.1</u>	<u>Performance in Safety Criteria</u>
	NUREG-0847 Supplement 0

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<u>Subsection</u>	<u>Title</u>
<u>4.4.2</u>	<u>Design Bases</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 12
<u>4.4.3</u>	<u>Thermal-Hydraulic Design Methodology</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 8
	NUREG-0847 Supplement 12
	NUREG-0847 Supplement 13
	NUREG-0847 Supplement 16
<u>4.4.4</u>	<u>Operating Abnormalities</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 13
<u>4.4.5</u>	<u>Loose Parts Monitoring System</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 3
	NUREG-0847 Supplement 5
	NUREG-0847 Supplement 16
<u>4.4.6</u>	<u>Thermal-Hydraulic Comparison</u>
	NUREG-0847 Supplement 0
<u>4.4.7</u>	<u>N-1 Loop Operation</u>
	NUREG-0847 Supplement 0
<u>4.4.8</u>	<u>Instrumentation for Inadequate Core Cooling Detection (II.F.2)</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 10
<u>4.4.9</u>	<u>Summary and Conclusion</u>
	NUREG-0847 Supplement 0
<u>4.5.1</u>	<u>Control Rod Drive Structural Materials</u>
	NUREG-0847 Supplement 0
<u>4.5.2</u>	<u>Reactor Internals and Core Support Materials</u>
	NUREG-0847 Supplement 0

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<u>Subsection</u>	<u>Title</u>
<u>4.6.0</u>	<u>Functional Design of Reactivity Control Systems</u> NUREG-0847 Supplement 0
<u>5.2.1</u>	<u>Compliance With Codes and Code Cases</u> NUREG-0847 Supplement 0
<u>5.2.2</u>	<u>Overpressure Protection</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 2
<u>5.2.3</u>	<u>Reactor Coolant Pressure Boundary Materials</u> NUREG-0847 Supplement 0
<u>5.2.4</u>	<u>RCS Pressure Boundary Inservice Inspection and Testing</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 12
<u>5.2.5</u>	<u>Reactor Coolant Pressure Boundary Leakage Detection</u> NUREG-0847 Supplement 0
<u>5.3.1</u>	<u>Reactor Vessel Materials</u> NUREG-0847 Supplement 0
<u>5.3.2</u>	<u>Pressure-Temperature Limits</u> NUREG-0847 Supplement 0
<u>5.3.3</u>	<u>Reactor Vessel Integrity</u> NUREG-0847 Supplement 0
<u>5.4.1</u>	<u>Reactor Coolant Pumps</u> NUREG-0847 Supplement 0
<u>5.4.2</u>	<u>Steam Generators</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 4
<u>5.4.3</u>	<u>Residual Heat Removal System</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 2 NUREG-0847 Supplement 10

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<u>Subsection</u>	<u>Title</u>
<u>5.4.4</u>	<u>Pressurizer Relief Tank</u> NUREG-0847 Supplement 0
<u>5.4.5</u>	<u>Reactor Coolant System Vents (II.B.1)</u> NUREG-0847 Supplement 0
<u>6.1.1</u>	<u>Metallic Materials</u> NUREG-0847 Supplement 0
<u>6.1.2</u>	<u>Organic Materials</u> NUREG-0847 Supplement 0
<u>6.1.3</u>	<u>Postaccident Emergency Cooling Water Chemistry</u> NUREG-0847 Supplement 0
<u>6.2.1</u>	<u>Containment Functional Design</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 3
<u>6.2.2</u>	<u>Containment Heat Removal Systems</u> NUREG-0847 Supplement 0
<u>6.2.3</u>	<u>Secondary Containment Functional Design</u> NUREG-0847 Supplement 0
<u>6.2.4</u>	<u>Containment Isolation System</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 3 NUREG-0847 Supplement 5 NUREG-0847 Supplement 12
<u>6.2.5</u>	<u>Combustible Gas Control Systems</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 4 NUREG-0847 Supplement 8
<u>6.2.6</u>	<u>Containment Leakage Testing</u> NUREG-0847 Supplement 0
<u>6.2.7</u>	<u>Fracture Prevention of Containment Pressure Boundary</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 4

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<u>Subsection</u>	<u>Title</u>
<u>6.3.0</u>	<u>Emergency Core Cooling System</u> NUREG-0847 Supplement 0
<u>6.3.1</u>	<u>System Design</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 7 NUREG-0847 Supplement 11
<u>6.3.2</u>	<u>Evaluation</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5
<u>6.3.3</u>	<u>Testing</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 2 NUREG-0847 Supplement 9
<u>6.3.4</u>	<u>Performance Evaluation</u> NUREG-0847 Supplement 0
<u>6.3.5</u>	<u>Conclusions</u> NUREG-0847 Supplement 0
<u>6.4.0</u>	<u>Control Room Habitability</u> NUREG-0847 Supplement 0
<u>6.5.1</u>	<u>ESF Atmosphere Cleanup Systems</u> NUREG-0847 Supplement 0
<u>6.5.2</u>	<u>Fission Product Cleanup System</u> NUREG-0847 Supplement 0
<u>6.5.3</u>	<u>Fission Product Control System</u> NUREG-0847 Supplement 0
<u>6.5.4</u>	<u>Ice Condenser as a Fission Product Control System</u> NUREG-0847 Supplement 0
<u>6.6.0</u>	<u>Inservice Inspection of Class 2 and 3 Components</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 10

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<u>Subsection</u>	<u>Title</u>
<u>7.1.1</u>	<u>General</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 13
	NUREG-0847 Supplement 16
<u>7.1.2</u>	<u>Comparison with Other Plants</u>
	NUREG-0847 Supplement 0
<u>7.1.3</u>	<u>Design Criteria</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 3
	NUREG-0847 Supplement 15
<u>7.2.0</u>	<u>Reactor Trip System</u>
	NUREG-0847 Supplement 0
<u>7.2.1</u>	<u>System Description</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 13
	NUREG-0847 Supplement 15
<u>7.2.2</u>	<u>Manual Trip Switches</u>
	NUREG-0847 Supplement 0
<u>7.2.3</u>	<u>Testing of Reactor Trip Breaker Shunt Coils</u>
	NUREG-0847 Supplement 0
<u>7.2.4</u>	<u>Anticipatory Trips</u>
	NUREG-0847 Supplement 0
<u>7.2.5</u>	<u>Steam Generator Water Level Trip</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 2
	NUREG-0847 Supplement 14
<u>7.2.6</u>	<u>Conclusions</u>
	NUREG-0847 Supplement 0
<u>7.3.0</u>	<u>Engineered Safety Features System</u>
	NUREG-0847 Supplement 0

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<u>Subsection</u>	<u>Title</u>
<u>7.3.1</u>	<u>System Description</u> NUREG-0847 Supplement 0
<u>7.3.2</u>	<u>Containment Sump Level Measurement</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 2
<u>7.3.3</u>	<u>Auxiliary Feedwater Initiation and Control</u> NUREG-0847 Supplement 0
<u>7.3.4</u>	<u>Failure Modes and Effects Analysis</u> NUREG-0847 Supplement 0
<u>7.3.5</u>	<u>IE Bulletin 80-06</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 3
<u>7.3.6</u>	<u>Conclusions</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 13
<u>7.4.1</u>	<u>System Description</u> NUREG-0847 Supplement 0
<u>7.4.2</u>	<u>Safe Shutdown from Auxiliary Control Room</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 7
<u>7.4.3</u>	<u>Conclusions</u> NUREG-0847 Supplement 0
<u>7.5.1</u>	<u>System Description</u> NUREG-0847 Supplement 0
<u>7.5.2</u>	<u>Post-Accident Monitoring System</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 9 NUREG-0847 Supplement 14 NUREG-0847 Supplement 15
<u>7.5.3</u>	<u>IE Bulletin 79-27</u> NUREG-0847 Supplement 0

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<u>Subsection</u>	<u>Title</u>
<u>7.5.4</u>	<u>Conclusions</u> NUREG-0847 Supplement 0
<u>7.6.1</u>	<u>System Description</u> NUREG-0847 Supplement 0
<u>7.6.2</u>	<u>Residual Heat Removal System Bypass Valves</u> NUREG-0847 Supplement 0
<u>7.6.3</u>	<u>Upper Head Injection Manual Control</u> NUREG-0847 Supplement 0
<u>7.6.4</u>	<u>Protection Against Spurious Actuation of Motor-Operated Valves</u> NUREG-0847 Supplement 0
<u>7.6.5</u>	<u>Overpressure Protection During Low Temperature Operation</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 4
<u>7.6.6</u>	<u>Valve Power Lockout</u> NUREG-0847 Supplement 0
<u>7.6.7</u>	<u>Cold Leg Accumulator Valve Interlocks and Position Indication</u> NUREG-0847 Supplement 0
<u>7.6.8</u>	<u>Automatic Switchover From Injection to Recirculation Mode</u> NUREG-0847 Supplement 0
<u>7.6.9</u>	<u>Conclusions</u> NUREG-0847 Supplement 0
<u>7.7.1</u>	<u>System Description</u> NUREG-0847 Supplement 0
<u>7.7.2</u>	<u>Safety System Status Monitoring System</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 7 NUREG-0847 Supplement 13
<u>7.7.3</u>	<u>Volume Control Tank Level Control System</u> NUREG-0847 Supplement 0

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<u>Subsection</u>	<u>Title</u>
<u>7.7.4</u>	<u>Pressurizer and Steam Generator Overfill</u> NUREG-0847 Supplement 0
<u>7.7.5</u>	<u>IE Information Notice 79-22</u> NUREG-0847 Supplement 0
<u>7.7.6</u>	<u>Multiple Control System Failures</u> NUREG-0847 Supplement 0
<u>7.7.7</u>	<u>Conclusions</u> NUREG-0847 Supplement 0
<u>7.7.8</u>	<u>Anticipated Transient Without Scram Mitigation System Actuation Circuitry (AMSAC)</u> NUREG-0847 Supplement 9 NUREG-0847 Supplement 14
<u>7.8.0</u>	<u>NUREG-0737 Items</u> NUREG-0847 Supplement 0
<u>7.8.1</u>	<u>Relief and Safety Valve Position Indication (II.D.3)</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5 NUREG-0847 Supplement 14
<u>7.8.2</u>	<u>Auxiliary Feedwater System Initiation and Flow Indication (II.E.1.2)</u> NUREG-0847 Supplement 0
<u>7.8.3</u>	<u>Proportional Integral Derivative Control Modification (II.K.3.9)</u> NUREG-0847 Supplement 0
<u>7.8.4</u>	<u>Proposed Anticipatory Trip Modification (II.K.3.10)</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 4
<u>7.8.5</u>	<u>Confirm Existence of Anticipatory Reactor Trip Upon Turbine Trip (II.K.3.12)</u> NUREG-0847 Supplement 0
<u>8.1.0</u>	<u>General</u> NUREG-0847 Supplement 0
<u>8.2.1</u>	<u>Compliance With GDC 5</u> NUREG-0847 Supplement 0

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<u>Subsection</u>	<u>Title</u>
<u>8.2.2</u>	<u>Compliance With GDC 17</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 2 NUREG-0847 Supplement 13
<u>8.2.3</u>	<u>Compliance With GDC 18</u> NUREG-0847 Supplement 0
<u>8.2.4</u>	<u>Evaluation Findings</u> NUREG-0847 Supplement 0
<u>8.3.0</u>	<u>Onsite Power Systems</u> NUREG-0847 Supplement 0
<u>8.3.1</u>	<u>Onsite AC Power System Compliance With GDC 17</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 2 NUREG-0847 Supplement 7 NUREG-0847 Supplement 13
<u>8.3.2</u>	<u>Onsite DC System Compliance With GDC 17</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 2 NUREG-0847 Supplement 13
<u>8.3.3</u>	<u>Evaluation Findings</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 2 NUREG-0847 Supplement 3 NUREG-0847 Supplement 7 NUREG-0847 Supplement 13
<u>9.0.0</u>	<u>Auxiliary Systems</u> NUREG-0847 Supplement 0
<u>9.1.1</u>	<u>New Fuel Storage</u> NUREG-0847 Supplement 0
<u>9.1.2</u>	<u>Spent Fuel Storage</u> NUREG-0847 Supplement 0

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<u>Subsection</u>	<u>Title</u>
<u>9.1.3</u>	<u>Spent Fuel Pool Cooling and Cleanup System</u> NUREG-0847 Supplement 0
<u>9.1.4</u>	<u>Fuel Handling System</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 13
<u>9.2.1</u>	<u>Essential Raw Cooling Water and Raw Cooling Water Systems</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 18
<u>9.2.2</u>	<u>Component Cooling System (Reactor Auxiliaries Cooling Water System)</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5
<u>9.2.3</u>	<u>Demineralized Water Makeup System</u> NUREG-0847 Supplement 0
<u>9.2.4</u>	<u>Potable and Sanitary Water Systems</u> NUREG-0847 Supplement 0
<u>9.2.5</u>	<u>Ultimate Heat Sink</u> NUREG-0847 Supplement 0
<u>9.2.6</u>	<u>Condensate Storage Facilities</u> NUREG-0847 Supplement 0
<u>9.3.1</u>	<u>Compressed Air System</u> NUREG-0847 Supplement 0
<u>9.3.2</u>	<u>Process Sampling System</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 3 NUREG-0847 Supplement 14
<u>9.3.3</u>	<u>Equipment and Floor Drainage System</u> NUREG-0847 Supplement 0
<u>9.3.4</u>	<u>Chemical and Volume Control System</u> NUREG-0847 Supplement 0

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<u>Subsection</u>	<u>Title</u>
<u>9.4.1</u>	<u>Control Room Area Ventilation System</u> NUREG-0847 Supplement 0
<u>9.4.2</u>	<u>Fuel Handling Area Ventilation System</u> NUREG-0847 Supplement 0
<u>9.4.3</u>	<u>Auxiliary and Radwates Area Ventilation System</u> NUREG-0847 Supplement 0
<u>9.4.4</u>	<u>Turbine Building Area Ventilation System</u> NUREG-0847 Supplement 0
<u>9.4.5</u>	<u>Engineered Safety Features Ventilation System</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 9 NUREG-0847 Supplement 10 NUREG-0847 Supplement 11 NUREG-0847 Supplement 16
<u>9.5.1</u>	<u>Fire Protection</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 18 NUREG-0847 Supplement 19
<u>9.5.2</u>	<u>Communication Systems</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5
<u>9.5.3</u>	<u>Lighting System</u> NUREG-0847 Supplement 0
<u>9.5.4</u>	<u>Emergency Diesel Engine Fuel Oil Storage and Transfer System</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5
<u>9.5.5</u>	<u>Emergency Diesel Engine Cooling Water System</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5

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<u>Subsection</u>	<u>Title</u>
<u>9.5.6</u>	<u>Emergency Diesel Engine Starting Systems</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5
<u>9.5.7</u>	<u>Emergency Diesel Engine Lubricating Oil System</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5
<u>9.5.8</u>	<u>Emergency Diesel Engine Combustion Air Intake and Exhaust System</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5
<u>10.1.0</u>	<u>Summary Description</u> NUREG-0847 Supplement 0
<u>10.2.1</u>	<u>Turbine Generator Design</u> NUREG-0847 Supplement 0
<u>10.2.2</u>	<u>Turbine Disc Integrity</u> NUREG-0847 Supplement 0
<u>10.3.0</u>	<u>Main Steam Supply System</u> NUREG-0847 Supplement 0
<u>10.3.1</u>	<u>Main Steam Supply System (up to and including the Main Steam Isolation Valves)</u> NUREG-0847 Supplement 0
<u>10.3.2</u>	<u>Main Steam Supply System</u> NUREG-0847 Supplement 0
<u>10.3.3</u>	<u>Steam and Feedwater System Materials</u> NUREG-0847 Supplement 0
<u>10.3.4</u>	<u>Secondary Water Chemistry</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5
<u>10.4.1</u>	<u>Main Condenser</u> NUREG-0847 Supplement 0
<u>10.4.2</u>	<u>Main Condenser Evacuation System</u> NUREG-0847 Supplement 0

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<u>Subsection</u>	<u>Title</u>
<u>10.4.3</u>	<u>Turbine Gland Sealing System</u> NUREG-0847 Supplement 0
<u>10.4.4</u>	<u>Turbine Bypass System</u> NUREG-0847 Supplement 0
<u>10.4.5</u>	<u>Condenser Circulating Water System</u> NUREG-0847 Supplement 0
<u>10.4.6</u>	<u>Condensate Cleanup System</u> NUREG-0847 Supplement 0
<u>10.4.7</u>	<u>Condensate and Feedwater Systems</u> NUREG-0847 Supplement 0
<u>10.4.8</u>	<u>Steam Generator Blowdown System</u> NUREG-0847 Supplement 0
<u>10.4.9</u>	<u>Auxiliary Feedwater System</u> NUREG-0847 Supplement 0
<u>11.1.0</u>	<u>Summary Description</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 16
<u>11.2.0</u>	<u>Liquid Waste Management</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 4 NUREG-0847 Supplement 16
<u>11.3.0</u>	<u>Gaseous Waste Management</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 8 NUREG-0847 Supplement 16
<u>11.4.0</u>	<u>Solid Waste Management System</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 16

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<u>Subsection</u>	<u>Title</u>
<u>11.5.0</u>	<u>Process and Effluent Radiological Monitoring and Sampling Systems</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 16 NUREG-0847 Supplement 20
<u>11.6.0</u>	<u>Evaluation Findings</u> NUREG-0847 Supplement 16
<u>11.7.0</u>	<u>NUREG-0737 Items</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5 NUREG-0847 Supplement 6
<u>11.7.1</u>	<u>Wide Range Noble Gas, Iodine, and Particulate Effluent Monitors (II.F.1(1) and II.F.1(2))</u> NUREG-0847 Supplement 5 NUREG-0847 Supplement 15
<u>11.7.2</u>	<u>Primary Coolant Outside Containment (III.D.1.1)</u> NUREG-0847 Supplement 5 NUREG-0847 Supplement 10 NUREG-0847 Supplement 16
<u>12.1.0</u>	<u>General</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 14
<u>12.2.0</u>	<u>Ensuring that Occupational Radiation Doses Are As Low As Reasonably Achievable</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 14
<u>12.3.0</u>	<u>Radiation Sources</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 14
<u>12.4.0</u>	<u>Radiation Protection Design Features</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 10 NUREG-0847 Supplement 14 NUREG-0847 Supplement 18

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<u>Subsection</u>	<u>Title</u>
<u>12.5.0</u>	<u>Dose Assessment</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 14
<u>12.6.0</u>	<u>Health Physics Program</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 10
	NUREG-0847 Supplement 14
<u>12.7.1</u>	<u>Plant Shielding (II.B.2)</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 14
	NUREG-0847 Supplement 16
<u>12.7.2</u>	<u>High Range Incontainment Monitor (II.F.1(3))</u>
	NUREG-0847 Supplement 5
<u>12.7.3</u>	<u>Inplant Radioiodine Monitor (III.D.3.3)</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 16
<u>13.1.1</u>	<u>Management and Technical Organization</u>
	NUREG-0847 Supplement 0
<u>13.1.2</u>	<u>Corporate Organization and Technical Support</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 8
<u>13.1.3</u>	<u>Plant Staff Organization</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 8
<u>13.2.1</u>	<u>Licensed Operator Training Program</u>
	NUREG-0847 Supplement 0
<u>13.2.2</u>	<u>Training for Nonlicensed Personnel</u>
	NUREG-0847 Supplement 0

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<u>Subsection</u>	<u>Title</u>
<u>13.3.1</u>	<u>Introduction</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 13
	NUREG-0847 Supplement 20
<u>13.3.2</u>	<u>Evaluation of the Emergency Plan</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 13
	NUREG-0847 Supplement 20
<u>13.3.3</u>	<u>Conclusions</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 13
	NUREG-0847 Supplement 20
<u>13.4.0</u>	<u>Review and Audit</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 8
<u>13.5.1</u>	<u>Administrative Procedures</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 9
	NUREG-0847 Supplement 10
<u>13.5.2</u>	<u>Operating and Maintenance Procedures</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 9
	NUREG-0847 Supplement 10
<u>13.5.3</u>	<u>NUREG-0737 Items</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 3
<u>13.6.0</u>	<u>Physical Security Plan</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 10
	NUREG-0847 Supplement 15

Table 7.
 NUREG-0847 and Supplemental Safety Evaluations Topic Inventory
 Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
<u>14.0.0</u>	<u>Initial Test Program</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 3
	NUREG-0847 Supplement 5
	NUREG-0847 Supplement 7
	NUREG-0847 Supplement 9
	NUREG-0847 Supplement 10
	NUREG-0847 Supplement 12
	NUREG-0847 Supplement 14
	NUREG-0847 Supplement 16
	NUREG-0847 Supplement 18
	NUREG-0847 Supplement 19
<u>15.1.0</u>	<u>General Discussion</u>
	NUREG-0847 Supplement 0
<u>15.2.0</u>	<u>Normal Operation and Anticipated Transients</u>
	NUREG-0847 Supplement 0
<u>15.2.1</u>	<u>Loss of Cooling Transients</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 13
	NUREG-0847 Supplement 14
<u>15.2.2</u>	<u>Increased Cooling Transients</u>
	NUREG-0847 Supplement 0
<u>15.2.3</u>	<u>Change in Coolant Inventory Transients</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 18
<u>15.2.4</u>	<u>Reactivity and Power Distribution Anomalies</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 4
	NUREG-0847 Supplement 7
	NUREG-0847 Supplement 13
	NUREG-0847 Supplement 14

Table 7.
NUREG-0847 and Supplemental Safety Evaluations Topic Inventory
Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
<u>15.2.5</u>	<u>Conclusions</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 4
<u>15.3.0</u>	<u>Limiting Accidents</u>
	NUREG-0847 Supplement 0
<u>15.3.1</u>	<u>Loss-of-Coolant Accident</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 12
	NUREG-0847 Supplement 15
<u>15.3.2</u>	<u>Steamline Break</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 14
<u>15.3.3</u>	<u>Feedwater System Pipe Break</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 14
<u>15.3.4</u>	<u>Reactor Coolant Pump Rotor Seizure</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 14
<u>15.3.5</u>	<u>Reactor Coolant Pump Shaft Break</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 14
<u>15.3.6</u>	<u>Anticipated Transients Without Scram</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 3
	NUREG-0847 Supplement 5
	NUREG-0847 Supplement 6
	NUREG-0847 Supplement 10
	NUREG-0847 Supplement 11
	NUREG-0847 Supplement 12
<u>15.3.7</u>	<u>Conclusions</u>
	NUREG-0847 Supplement 0

Table 7.
 NUREG-0847 and Supplemental Safety Evaluations Topic Inventory
 Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
<u>15.4.0</u>	<u>Radiological Consequences of Accidents</u> NUREG-0847 Supplement 0
<u>15.4.1</u>	<u>Loss-of-Coolant Accident</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5 NUREG-0847 Supplement 9 NUREG-0847 Supplement 15 NUREG-0847 Supplement 18
<u>15.4.2</u>	<u>Main Steamline Break Outside of Containment</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 18
<u>15.4.3</u>	<u>Steam Generator Tube Rupture</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 3 NUREG-0847 Supplement 5 NUREG-0847 Supplement 12 NUREG-0847 Supplement 14 NUREG-0847 Supplement 15
<u>15.4.4</u>	<u>Control Rod Ejection Accident</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 15
<u>15.4.5</u>	<u>Fuel-Handling Accident</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 4 NUREG-0847 Supplement 15
<u>15.4.6</u>	<u>Failure of Small Line Carrying Coolant Outside Containment</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 15
<u>15.4.7</u>	<u>Postulated Radioactive Releases as a Result of Liquid Tank Failures</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 15

Table 7.
NUREG-0847 and Supplemental Safety Evaluations Topic Inventory
Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
<u>15.5.1</u>	<u>Thermal Mechanical Report (II.K.2.13)</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 4
<u>15.5.2</u>	<u>Voiding in the Reactor Coolant System During Transients (II.K.2.17)</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 4
<u>15.5.3</u>	<u>Installation and Testing of Automatic Power-Operated Relief Valve Isolation System (II.K.3.1)</u> <u>Report on Overall Safety Effect of Power-Operated Relief Valve Isolation System (II.K.3.2)</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5
<u>15.5.4</u>	<u>Automatic Trip of Reactor Coolant Pumps (II.K.3.5)</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 4 NUREG-0847 Supplement 5 NUREG-0847 Supplement 16
<u>15.5.5</u>	<u>Small-Break LOCA Methods (II.K.3.30) and Plant-Specific Calculations (II.K.3.31)</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 4 NUREG-0847 Supplement 5 NUREG-0847 Supplement 16
<u>15.6.0</u>	<u>Relative Risk of Low Power Operation</u> NUREG-0847 Supplement 0
<u>16.0.0</u>	<u>Technical Specifications</u> NUREG-0847 Supplement 0
<u>17.1.0</u>	<u>General</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5
<u>17.2.0</u>	<u>Organization</u> NUREG-0847 Supplement 0 NUREG-0847 Supplement 5 NUREG-0847 Supplement 13

Table 7.
NUREG-0847 and Supplemental Safety Evaluations Topic Inventory
Watts Bar Nuclear Plant Unit 2

<u>Subsection</u>	<u>Title</u>
<u>17.3.0</u>	<u>Quality Assurance Program</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 2
	NUREG-0847 Supplement 5
	NUREG-0847 Supplement 10
	NUREG-0847 Supplement 13
	NUREG-0847 Supplement 15
<u>17.4.0</u>	<u>Conclusions</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 2
	NUREG-0847 Supplement 5
	NUREG-0847 Supplement 13
<u>18.1.0</u>	<u>General</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 16
<u>18.2.0</u>	<u>Conclusions</u>
	NUREG-0847 Supplement 0
	NUREG-0847 Supplement 16
<u>20.0.0</u>	<u>Common Defense and Security</u>
	NUREG-0847 Supplement 0
<u>21.0.0</u>	<u>Financial Qualifications</u>
	NUREG-0847 Supplement 0
<u>22.1.0</u>	<u>General</u>
	NUREG-0847 Supplement 0
<u>22.2.0</u>	<u>Preoperational Storage of Nuclear Fuel</u>
	NUREG-0847 Supplement 0
<u>22.3.0</u>	<u>Operating Licenses</u>
	NUREG-0847 Supplement 0

Table 8.
NUREG-0847 Supplements Not Addressed By TVA

<u>Subsection</u>	<u>Title</u>
3.2.1	Seismic Qualification Does not address SSER 6 and 8
3.2.2	System Quality Group Classification Does not address SSER 3, 6, 7, and 9
3.5.1	Missile Selection and Description Does not address SSER 9 and 14
3.6.2	Determination of Break Locations and Dynamic Effects Associated with the Postulated Rupture of Piping Does not address SSER 14
3.7.1	Seismic Input Does not address SSER 9, and 16
3.7.2	Seismic Analysis Does not address SSER 16
3.8.1	Steel Containment Not addressed in SSER 9
3.9.2	Dynamic Testing and Analysis of Systems, Components, and Equipment Does not address SSER 14
3.9.3	ASME Code Class 1, 2, and 3 Components, Component Structures, and Core Support Structures Does not address SSER 15
3.9.6	Inservice Testing of Pumps and Valves Does not address SSER 20
4.2.1	Description Does not address SSER 13
4.2.3	Mechanical Performance Does not address SSER 10 and 13
4.3.2	Design Description Does not address SSER 13 and 15
4.3.3	Analytical Methods Does not address SSER 13
4.3.4	Summary of Evaluation Findings Does not address SSER 13

Table 8, continued.
NUREG-0847 Supplements Not Addressed By TVA

<u>Subsection</u>	<u>Title</u>
4.4.2	Design Bases Does not address SSER 12
4.4.3	Thermal-Hydraulic Design Methodology Does not address SSER 8, 12, 13, and 16
5.2.5	Reactor Coolant Pressure Boundary Leakage Detection Does not address SSER 9, 11, and 12
5.4.3	Residual Heat Removal System Does not address SSER 10
6.2.4	Containment Isolation System Does not address SSER 12
6.2.5	Combustible Gas Control Systems Does not address SSER 4 and 8
6.3.1	System Design Does not address SSER 11
6.3.2	Evaluation Does not address SSER 5
7.1.1	General Does not address SSER 13 and 16
7.2.1	System Description Does not address SSER 13 and 15
7.3.6	Conclusions Does not address SSER 13
7.4.2	Safe Shutdown from Auxiliary Control Room Does not address SSER 7
7.7.2	Safety System Status Monitoring System Does not address SSER 13
7.7.8	Anticipated Transient Without Scram Mitigation System Actuation Circuitry (AMSAC) Does not address SSER 14
7.8.1	Relief and Safety Valve Position Indication (II.D.3) Does not address SSER 5 and 14
8.3.1	Onsite AC Power System Compliance With GDC 17 Does not address SSER 13

Table 8, continued.
NUREG-0847 Supplements Not Addressed By TVA

<u>Subsection</u>	<u>Title</u>
9.1.2	Spent Fuel Storage Does not address SSER 5, 15, and 16
9.2.4	Potable and Sanitary Water Systems Does not address SSER 9
9.2.6	Condensate Storage Facilities Does not address SSER 12
9.3.2	Process Sampling System Does not address SSER 16
9.5.4	Emergency Diesel Engine Fuel Oil Storage and Transfer System Does not address SSER 9, 10, 11, and 12
9.5.5	Emergency Diesel Engine Cooling Water System Does not address SSER 11
9.5.6	Emergency Diesel Engine Starting Systems Does not address SSER 10
9.5.7	Emergency Diesel Engine Lubricating Oil System Does not address SSER 10
9.5.8	Emergency Diesel Engine Combustion Air Intake and Exhaust System Does not address SSER 10
10.2.0	Turbine Generator Does not address SSER 5
10.2.1	Turbine Generator Design Does not address SSER 12
10.3.1	Main Steam Supply System (up to and including the Main Steam Isolation Valves) Does not address SSER 12 and 19
10.4.1	Main Condenser Does not address SSER 9
10.4.4	Turbine Bypass System Does not address SSER 5
10.4.7	Condensate and Feedwater Systems Does not address SSER 14
10.4.9	Auxiliary Feedwater System Does not address SSER 14

Table 8, continued.
NUREG-0847 Supplements Not Addressed By TVA

<u>Subsection</u>	<u>Title</u>
11.7.0	NUREG-0737 Items Does not address SSER 5 or 6
13.5.2	Operating and Maintenance Procedures Does not address SSER 10
14.0.0	Initial Test Program Does not address SSER 16, 18, and 19
15.2.1	Loss of Cooling Transients Does not address SSER 13 and 14
15.2.3	Change in Coolant Inventory Transients Does not address SSER 18
15.2.4	Reactivity and Power Distribution Anomalies Does not address SSER 7, 13, and 14
15.3.1	Loss-of-Coolant Accident Does not address SSER 12 and 15
15.3.2	Steamline Break Does not address SSER 14
15.3.3	Feedwater System Pipe Break Does not address SSER 14
15.3.4	Reactor Coolant Pump Rotor Seizure Does not address SSER 14
15.3.5	Reactor Coolant Pump Shaft Break Does not address SSER 14
15.3.6	Anticipated Transients Without Scram Does not address SSER 6, 10, 11, 12
15.4.1	Loss-of-Coolant Accident Does not address SSER 5, 9, 15, and 18
15.4.2	Main Steamline Break Outside of Containment Does not address SSER 18
15.4.3	Steam Generator Tube Rupture Does not address SSER 15
15.4.4	Control Rod Ejection Accident Does not address SSER 15

Table 8, continued.
NUREG-0847 Supplements Not Addressed By TVA

<u>Subsection</u>	<u>Title</u>
15.4.5	Fuel-Handling Accident Does not address SSER 4 and 15
15.4.6	Failure of Small Line Carrying Coolant Outside Containment Does not address SSER 15
15.4.7	Postulated Radioactive Releases as a Result of Liquid Tank Failures Does not address SSER 15
17.1.0	General Does not address SSER 5
17.3.0	Quality Assurance Program Does not address SSER 2, 5, 10, 13, and 15
17.4.0	Conclusions Does not address SSER 2, 5, and 13