

R3

From: "RLS4@NRC.GOV" <rls4@nrc.gov>
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Date: 04/05/2007 7:44:35 AM
Subject: New CR on Eexponent report & DB actions

CC: "JER7@NRC.GOV" <jer7@nrc.gov>, "RLS4@NRC.GOV" <rls4@nrc.gov>

F-120

Mail Envelope Properties (4614EF04.50C : 7 : 38156)

Subject: New CR on Eexponent report & DB actions
Creation Date 04/05/2007 7:43:48 AM
From: "RLS4@NRC.GOV" <rls4@nrc.gov>
Created By: rls4@nrc.gov

Recipients

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ch_po.CH_DO

ERD (Eric Duncan)

GCW (Geoffrey Wright)

JER7 CC (John Rutkowski)

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TWGWPO04.HQGWDO01

TJW2 (Thomas Wengert)

Post Office

ch_po.CH_DO

TWGWPO04.HQGWDO01

Route

nrc.gov

nrc.gov

Files

MESSAGE

Document.pdf

Mime.822

Size

2

417156

572671

Date & Time

04/05/2007 7:43:48 AM

Options**Expiration Date:**

None

Priority:

Standard

ReplyRequested:

No

Return Notification:

None

Concealed Subject:

No

Security:

Standard

Junk Mail Handling Evaluation Results

Message is eligible for Junk Mail handling

This message was not classified as Junk Mail

Junk Mail settings when this message was delivered

Junk Mail handling disabled by User

Junk Mail handling disabled by Administrator
Junk List is not enabled
Junk Mail using personal address books is not enabled
Block List is not enabled

CONDITION REPORT

CR Number
 07-17452

TITLE: EXPONENT FAILURE ANALYSIS REPORT OF THE DB RPV HEAD DISCOVERED MARCH 2002

O R I G I N A T I O N	DISCOVERY DATE	TIME	EVENT DATE	TIME	SYSTEM / ASSET#
	2/23/2007	N/A	N/A	N/A	
	EQUIPMENT DESCRIPTION REACTOR VESSEL 1-1				
	FLOC System			FLOC	
	DESCRIPTION OF CONDITION and PROBABLE CAUSE (if known) Summarize any attachments. Identify what, when, where, why, how. FirstEnergy Nuclear Operating Company (FENOC) received a technical report, dated December 15, 2006, entitled "Review & Analysis of the Davis-Besse March 2002 Reactor Pressure Vessel Head Wastage Event", from Exponent Failure Analysis Associates and Altran Solutions Corporation. This report was developed in support of a recent insurance claim for the reactor pressure vessel head event. The report draws upon existing FENOC Root Cause Evaluations, subsequent analysis of the specimens removed from the reactor vessel head, as well as industry materials test program results subsequent to the Root Cause Evaluations. The Report states in part, that susceptible materials can have crack growth rates that are significantly higher than previously assumed, and small through wall cracks can lead to high rates of erosion and corrosion. As a result of the information received, the Root Cause Evaluations performed under Condition Report 2002-00891, entitled Control Rod Drive Nozzle Crack Indication, will be reviewed under this Condition Report to determine if the corrective actions taken as a result of these root causes, performed as a result of the initial RPV head leakage, bound the conclusions reflected in the report by Exponent Failure Analysis Associates.				
	IMMEDIATE ACTIONS TAKEN / SUPV COMMENTS (Discuss CORRECTIVE ACTIONS completed, basis for closure.) This CR is being generated to perform and document a comparative review of information contained in the Exponent Report against the original root cause analyses for the reactor pressure vessel head degradation discovered during the 13th refueling outage in March 2002. CR 07-15077 was previously initiated to address a concern raised by Nuclear Electric Insurance Limited (NEIL) as to whether the Exponent report identified a new safety concern based on information contained in the Exponent report.				
QUALITY ORGANIZATION USE ONLY Quality Org. Initiated ☐ Yes Quality Org. Follow-up ☐ Yes ☐ No		IDENTIFIED BY (Check one) ☐ Individual/Work Group ☒ Supervision/Management		☐ Self-Revealed ☐ Internal Oversight ☐ External Oversight	
ORIGINATOR HENNESSY, B		ORGANIZATION DBRC	DATE 4/3/2007	SUPERVISOR PRICE, C	DATE 4/3/2007
					PHONE EXT. 8585

CONDITION REPORT

CR Number
 07-17452

TITLE: EXPONENT FAILURE ANALYSIS REPORT OF THE DB RPV HEAD DISCOVERED MARCH 2002

P L A N T O P E R A T I O N S	SRO REVIEW	EQUIPMENT OPERABLE	OPERABILITY ASSESSMENT REQUIRED	ORG. NOTIFIED	IMMEDIATE INVESTIGATION REQUIRED	ORG. NOTIFIED	MODE CHANGE RESTRAINT	
	☒ Yes ☐ No	☐ Yes ☐ No ☒ N/A	☐ Yes ☒ No		☐ Yes ☒ No		☐ Yes ☒ No	
	MODE	ASSOCIATED TECH SPEC NUMBER(S)	ASSOCIATED LCO ACTION STATEMENT(S)					
	N/A	N/A	#1 N/A					
			#2					
			#3					
	DECLARED INOPERABLE (Date / Time)	REPORTABLE?	One Hour N/A			APPLICABLE UNIT(S)		
	N/A	☐ Yes ☒ No	Four Hour N/A			☒ U1 ☐ U2 ☐ Both		
		☐ Eval Required	Eight Hour N/A					
			Other N/A					
COMMENTS								
This CR is to document a review that will be done comparing a previous root cause done in 2002 to more recent research which contains additional testing done since 2002. This is an administrative issue. If during this review a condition is identified that is not bounded by the 2002 root cause another CR will need to be written to assess that finding for applicability and operability. Equip Operable? N/A and Reportable is no.								
Current Mode - Unit 1		Power Level - Unit 1	Current Mode - Unit 2	Power Level - Unit 2				
1		100	N/A		N/A			
SRO - UNIT 1			SRO - UNIT 2			DATE		
Cuff, J			Gore, K			4/4/2007		
CRPA / SUPV / MRB	CATEGORY / EVAL	ASSIGNED ORGANIZATION	DUE DATE	R E G U L A T O R Y	REPORTABLE?			
	CF	FDRA			☐ Yes ☐ No ☐ LER No.			
	TREND CODES		Comp Type / ID		Cause	REPORTABILITY REVIEWER		
	Process / Activity / Cause Code(s)		(If Cause T or W)		Org			
	LP2 0960							
INVESTIGATION OPTIONS				CLOSED BY	DATE			
☐ Maint.Rule ☐ OE Evaluation								