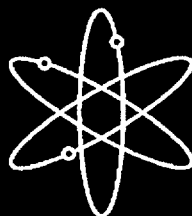




Generic Environmental Impact Statement for License Renewal of Nuclear Plants



Supplement 8



**Regarding
McGuire Nuclear Station, Units 1 and 2**



Draft Report for Comment



**U.S. Nuclear Regulatory Commission
Office of Nuclear Reactor Regulation
Washington, DC 20555-0001**



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Generic Environmental Impact Statement for License Renewal of Nuclear Plants

Supplement 8

Regarding McGuire Nuclear Station, Units 1 and 2

Draft Report for Comment

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**Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001**



COMMENTS ON DRAFT REPORT

Any interested party may submit comments on this report for consideration by the NRC staff. Comments may be accompanied by additional relevant information or supporting data. Please specify the report number NUREG-1437, Supplement 8, draft, in your comments, and send them by August 2, 2002 to the following address:

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Electronic comments may be submitted to the NRC by the Internet at McGuireEIS@nrc.gov.

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Abstract

The U.S. Nuclear Regulatory Commission (NRC) considered the environmental impacts of renewing nuclear power plant operating licenses (OLs) for a 20-year period in its *Generic Environmental Impact Statement for License Renewal of Nuclear Plants* (GEIS), NUREG-1437, Volumes 1 and 2, and codified the results in 10 CFR Part 51. The GEIS (and its Addendum 1) identifies 92 environmental issues and reaches generic conclusions related to environmental impacts for 69 of these issues that apply to all plants or to plants with specific design or site characteristics. Additional plant-specific review is required for the remaining 23 issues. These plant-specific reviews are to be included in a supplement to the GEIS.

This draft Supplemental Environmental Impact Statement (SEIS) has been prepared in response to an application submitted to the NRC by Duke Energy Corporation (Duke) to renew the OLs for McGuire Nuclear Station, Units 1 and 2 (McGuire) for an additional 20 years under 10 CFR Part 54. This draft SEIS includes the NRC staff's analysis that considers and weighs the environmental impacts of the proposed action, the environmental impacts of alternatives to the proposed action, and mitigation measures available for reducing or avoiding adverse impacts. It also includes the staff's preliminary recommendation regarding the proposed action.

Regarding the 69 issues for which the GEIS reached generic conclusions, neither Duke nor the staff has identified information that is both new and significant for any of these issues that apply to McGuire. In addition, the staff determined that information provided during the scoping process did not call into question the conclusions in the GEIS. Therefore, the staff concludes that the impacts of renewing the McGuire OLs will not be greater than impacts identified for these issues in the GEIS. For each of these issues, the GEIS conclusion is that the impact is of SMALL^(a) significance (except for collective offsite radiological impacts from the fuel cycle and high-level waste and spent fuel, which were not assigned single significance levels).

Regarding the remaining 23 issues, those that apply to McGuire are addressed in this draft SEIS. For each applicable issue, the staff concludes that the significance of the potential environmental impacts of renewal of the OLs is SMALL. The staff also concludes that additional mitigation measures are not likely to be sufficiently beneficial as to be warranted. The staff determined that information provided during the scoping process did not identify any new issue that has a significant environmental impact.

The NRC staff's preliminary recommendation is that the Commission determine that the adverse environmental impacts of license renewal for McGuire are not so great that preserving the option of license renewal for energy-planning decisionmakers would be unreasonable. This recommendation is based on (1) the analysis and findings in the GEIS; (2) the Environmental

(a) Environmental effects are not detectable or are so minor that they neither destabilize nor noticeably alter any important attribute of the resource.

Abstract

- 1 Report submitted by Duke; (3) consultation with Federal, State, and local agencies; (4) the
- 2 staff's own independent review, and (5) the staff's consideration of public comments received
- 3 during the scoping process.

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Executive Summary

By letter dated June 13, 2001, Duke Energy Corporation (Duke) submitted an application to the U.S. Nuclear Regulatory Commission (NRC) to renew the operating licenses (OLs) for McGuire Nuclear Station, Units 1 and 2 (McGuire) for up to an additional 20-year period. If the OLs are renewed, State regulatory agencies and Duke will ultimately decide whether the plant will continue to operate based on factors such as the need for power or other matters within the State's jurisdiction or the purview of the owners. If the OLs are not renewed, the plant must be shut down at or before the expiration dates of the current OLs, which are June 12, 2021, for Unit 1, and March 3, 2023, for Unit 2.

Section 102 of the National Environmental Policy Act (NEPA) (42 USC 4321) directs that an environmental impact statement (EIS) is required for major Federal actions that significantly affect the quality of the human environment. The NRC has implemented Section 102 of NEPA in 10 CFR Part 51. Part 51 identifies licensing and regulatory actions that require an EIS. In 10 CFR 51.20(b)(2), the Commission requires preparation of an EIS or a supplement to an EIS for renewal of a reactor OL; 10 CFR 51.95(c) states that the EIS prepared at the OL renewal stage will be a supplement to the *Generic Environmental Impact Statement for License Renewal of Nuclear Plants* (GEIS), NUREG-1437, Volumes 1 and 2 (NRC 1996, 1999).^(a)

Upon acceptance of the Duke application, the NRC began the environmental review process described in 10 CFR Part 51 by publishing a notice of intent to prepare an EIS and conduct scoping. The staff visited the McGuire site in September 2001 and held public scoping meetings on September 25, 2001, in Huntersville, North Carolina. In preparing this draft Supplemental Environmental Impact Statement (SEIS) for McGuire, the staff reviewed the McGuire Environmental Report (ER) and compared it to the GEIS, consulted with other agencies, conducted an independent review of the issues following the guidance set forth in NUREG-1555, Supplement 1, the *Standard Review Plans for Environmental Reviews for Nuclear Power Plants, Supplement 1: Operating License Renewal*, and considered the public comments received during the scoping process. The public comments received during the scoping process that were considered to be within scope of the environmental review are provided in Appendix A, Part 1, of this SEIS.

The staff will hold two public meetings in the vicinity of the McGuire site in June 2002 to describe the preliminary results of the NRC environmental review, to answer questions, and to provide members of the public with information to assist them in formulating comments on this SEIS. When the comment period ends, the staff will consider and disposition all of the comments received. These comments will be addressed in Appendix A, Part 2, of the SEIS.

(a) The GEIS was originally issued in 1996. Addendum 1 to the GEIS was issued in 1999. Hereafter, all references to the "GEIS" include the GEIS and its Addendum 1.

Executive Summary

1 This SEIS includes the staff's preliminary analysis that considers and weighs the environmental
2 effects of the proposed action, the environmental impacts of alternatives to the proposed action,
3 and mitigation measures for reducing or avoiding adverse effects. It also includes the staff's
4 preliminary recommendation regarding the proposed action.

5
6 The Commission has adopted the following statement of purpose and need for license renewal
7 from the GEIS:

8
9 The purpose and need for the proposed action (renewal of an operating license)
10 is to provide an option that allows for power generation capability beyond the
11 term of a current nuclear power plant operating license to meet future system
12 generating needs, as such needs may be determined by State, utility, and, where
13 authorized, Federal (other than NRC) decisionmakers.

14
15 The goal of the staff's environmental review, as defined in 10 CFR 51.95(c)(4) and the GEIS, is
16 to determine

17
18 ... whether or not the adverse environmental impacts of license renewal are so
19 great that preserving the option of license renewal for energy planning
20 decisionmakers would be unreasonable.

21
22 Both the statement of purpose and need and the evaluation criterion implicitly acknowledge that
23 there are factors, in addition to license renewal, that will ultimately determine whether an
24 existing nuclear power plant continues to operate beyond the period of the current OLs.

25
26 NRC regulations [10 CFR 51.95(c)(2)] contain the following statement regarding the content of
27 SEISs prepared at the license renewal stage:

28
29 The supplemental environmental impact statement for license renewal is not
30 required to include discussion of need for power or the economic costs and
31 economic benefits of the proposed action or of alternatives to the proposed
32 action except insofar as such benefits and costs are either essential for a
33 determination regarding the inclusion of an alternative in the range of
34 alternatives considered or relevant to mitigation. In addition, the supplemental
35 environmental impact statement prepared at the license renewal stage need not
36 discuss other issues not related to the environmental effects of the proposed
37 action and the alternatives, or any aspect of the storage of spent fuel for the
38 facility within the scope of the generic determination in § 51.23(a) ["Temporary
39 storage of spent fuel after cessation of reactor operations—generic determination
40 of no significant environmental impact"] and in accordance with § 51.23(b).

The GEIS contains the results of a systematic evaluation of the consequences of renewing an OL and operating a nuclear power plant for an additional 20 years. In the GEIS, the staff evaluated 92 environmental issues using the NRC's three-level standard of significance—SMALL, MODERATE, or LARGE—developed using the Council on Environmental Quality guidelines. The following definitions of the three significance levels are set forth in footnotes to Table B-1 of 10 CFR Part 51, Subpart A, Appendix B:

SMALL - Environmental effects are not detectable or are so minor that they will neither destabilize nor noticeably alter any important attribute of the resources.

MODERATE - Environmental effects are sufficient to alter noticeably, but not to destabilize, important attributes of the resource.

LARGE - Environmental effects are clearly noticeable and are sufficient to destabilize important attributes of the resource.

For 69 of the 92 issues considered in the GEIS, the analysis in the GEIS led to the following conclusions:

- (1) The environmental impacts associated with the issue have been determined to apply either to all plants or, for some issues, to plants having a specific type of cooling system or other specified plant or site characteristics.
- (2) A single significance level (i.e., SMALL, MODERATE, or LARGE) has been assigned to the impacts (except for collective offsite radiological impacts from the fuel cycle and from high-level waste and spent fuel disposal).
- (3) Mitigation of adverse impacts associated with the issue has been considered in the analysis, and it has been determined that additional plant-specific mitigation measures are likely not to be sufficiently beneficial to warrant implementation.

These 69 issues were identified in the GEIS as Category 1 issues. In the absence of new and significant information, the staff relied on conclusions as amplified by supporting information in the GEIS for issues designated Category 1 in Table B-1 of 10 CFR Part 51, Subpart A, Appendix B.

Of the 23 issues that do not meet the criteria set forth above, 21 are classified as Category 2 issues requiring analysis in a plant-specific supplement to the GEIS. The remaining two issues, environmental justice and chronic effects of electromagnetic fields, were not categorized. Environmental justice was not evaluated on a generic basis and must be addressed in a plant-

Executive Summary

1 specific supplement to the GEIS. Information on the chronic effects of electromagnetic fields
2 was not conclusive at the time the GEIS was prepared.

3
4 This SEIS documents the staff's evaluation of all 92 environmental issues considered in the
5 GEIS. The staff considered the environmental impacts associated with alternatives to license
6 renewal and compared the environmental impacts of license renewal and the alternatives. The
7 alternatives to license renewal that were considered include the no-action alternative (not
8 renewing the OLs for McGuire, Units 1 and 2) and alternative methods of power generation.
9 Based on projections made by the U.S. Department of Energy's Energy Information
10 Administration, gas- and coal-fired generation appear to be the most likely power-generation
11 alternatives if the power from Units 1 and 2 is replaced. These alternatives are evaluated
12 assuming that the replacement power generation plant is located at either the McGuire site or
13 some other unspecified location.

14
15 Mitigation measures were considered for each Category 2 issue. Current measures to mitigate
16 the environmental impacts of plant operation were found to be adequate, and no additional
17 mitigation measures were deemed sufficiently beneficial to be warranted.

18
19 If the McGuire OLs are not renewed and the units cease operation on or before the expiration
20 of their current OLs, then the adverse impacts of likely alternatives will not be smaller than
21 those associated with continued operation of McGuire. The impacts may, in fact, be greater in
22 some areas.

23
24 The preliminary recommendation of the NRC staff is that the Commission determine that the
25 adverse environmental impacts of license renewal for McGuire are not so great that preserving
26 the option of license renewal for energy planning decisionmakers would be unreasonable. This
27 recommendation is based on (1) the analysis and findings in the GEIS; (2) the ER submitted by
28 Duke; (3) consultation with other Federal, State, and local agencies; (4) the staff's own
29 independent review; and (5) the staff's consideration of public comments received during the
30 scoping process.

Abbreviations/Acronyms

1		
2		
3		
4	°	degree
5	μm	micrometer
6	μCi	microcurie
7		
8	AADT	Annual Average Daily Traffic
9	ac	acre
10	ac.	Alternating current
11	ACC	averted cleanup and decontamination costs
12	AEA	Atomic Energy Act
13	AEC	Atomic Energy Commission
14	AOC	averted offsite property damage costs
15	AOE	averted occupational exposure
16	AOSC	averted onsite costs
17	APE	averted public exposure
18	APRC	averted power replacement cost
19	ATWS	anticipated transient without scram
20		
21	Bq	becquerel
22	Btu	British thermal unit
23	Btu/kWh	British thermal units per kilowatt hour
24	Btu/lb	British thermal units per pound
25	BWR	boiling water reactor
26		
27	C	Celsius
28	CAA	Clean Air Act
29	CDC	Center for Disease Control and Prevention
30	CDF	core damage frequency
31	CEQ	Council on Environmental Quality
32	CET	containment event tree
33	CFR	Code of Federal Regulations
34	Ci	curie
35	CMUD	Charlotte-Mecklenburg Utilities District
36	COE	Cost of enhancement
37	CWA	Clean Water Act
38		
39	DBA	design-basis accident
40	DCH	direct containment heating
41	DG	diesel generator
42	DOE	U.S. Department of Energy
43	DSM	demand-side management

Abbreviations/Acronyms

1	Duke	Duke Energy Corporation
2		
3	E	endangered
4	ECCS	emergency core cooling system
5	EIA	Energy Information Agency
6	EIS	Environmental Impact Statement
7	ELF	extremely low frequency
8	EMF	electromagnetic field
9	EPA	U.S. Environmental Protection Agency
10	EPZ	Emergency Planning Zone
11	ER	Environmental Report
12	ESRP	Environmental Standard Review Plan
13	EX	extirpated
14		
15	F	Fahrenheit
16	FAA	Federal Aviation Administration
17	FERC	Federal Energy Regulatory Commission
18	FES	Final Environmental Statement
19	FR	Federal Register
20	FSAR	Final Safety Analysis Report
21	FSC	Federal species of concern
22	ft	feet
23	ft/s	feet per second
24	ft ³	cubic feet
25	F-V	Fussell-Vesely
26	FWPCA	Federal Water Pollution Control Act
27	FWS	U. S. Fish and Wildlife Service
28	FWST	refueling water storage tank
29		
30	gal	gallon
31	GEIS	Generic Environmental Impact Statement
32	gpd	gallons per day
33	gpm	gallons per minute
34	GSI	Generic Safety Issue
35		
36	ha	hectare
37	HEPA	high-efficiency particulate air (filter)
38	HLW	high-level waste
39	hr	hour(s)
40	Hz	hertz

Abbreviations/Acronyms

1	I&C	instrumentation and control
2	IBA	Important Bird Area
3	IEEE	Institution of Electrical and Electronic Engineers
4	IPE	individual plant examination
5	IPEEE	individual plant examination for external events
6	ISFSI	Independent Spent Fuel Storage Installation
7	ISLOCA	interfacing loss of coolant accident
8		
9	J	joule
10		
11	km	kilometer
12	kV	kilovolt
13	kWh	kilowatt-hour
14		
15	L	liter
16	L/s	liters per second
17	LNG	liquefied natural gas
18	LOCA	loss-of-coolant accident
19	LOOP	loss of offsite power
20	LOS	level of service
21	LWR	light-water reactor
22		
23	m	meter
24	m/s	meter per second
25	m ³	cubic meter
26	m ³ /d	cubic meter per day
27	MAAP	Modular Accident Analysis Program
28	MACCS2	MELCOR Accident Consequence Code System 2
29	McGuire	McGuire Nuclear Station
30	mgd	million gallons per day
31	mGy	milligray
32	mi	mile
33	MJ/kg	million joules per kilogram
34	mL	milliliter
35	mph	miles per hour
36	mrاد	millirad
37	mrem	millirem
38	mSv	millisievert
39	MT	metric ton
40	MTHM	metric tonnes of heavy metal (uranium, etc.)
41	MUMPO	Mecklenburg-Union Metropolitan Planning Organization

Abbreviations/Acronyms

1	MW	megawatt
2	MW(e)	megawatts electric
3	MW(t)	megawatts thermal
4	MWd/MTU	megawatt days per metric ton uranium
5	MWh	megawatt hour
6		
7	NA	not applicable
8	NAS	National Academy of Sciences
9	NC	North Carolina
10	NCDCR	North Carolina Department of Cultural Resources
11	NCDENR	North Carolina Department of Environmental and Natural Resources
12	NCDHHS	North Carolina Department of Health and Human Services
13	NCDNRCD	North Carolina Department of Natural Resources and Community Development
14	NCDOT	North Carolina Department of Transportation
15	NCWRC	North Carolina Wildlife Resource Commission
16	NEPA	National Environmental Policy Act
17	NESC	National Electrical Safety Code
18	ng/J	nanograms per joule
19	NHPA	National Historic Preservation Act
20	NIEHS	National Institute of Environmental Health Sciences
21	NO ₂	nitrogen dioxide
22	NO _x	nitrogen oxide
23	NPDES	National Pollutant Discharge Elimination System
24	NRC	U.S. Nuclear Regulatory Commission
25	NRR	Nuclear Reactor Regulation
26	NWPPC	Northwest Power Planning Council
27		
28	ODCM	Offsite Dose Calculation Manual
29	OL	operating license
30		
31	PAME	primary amebic meningoencephalitis
32	PAR	passive autocatalytic recombiner
33	PDS	plant damage state
34	PM	particulate matter
35	PM ₁₀	particulate matter having aerodynamic diameter less than or equal to 10 μ m
36	PRA	Probabilistic Risk Assessment
37	PSD	prevention of significant deterioration
38	PW	present worth
39	PWR	pressurized water reactor
40	PU _{RP}	present valve replacement power cost

Abbreviations/Acronyms

1	RAI	request for additional information
2	RCRA	Resource Conservation and Recovery Act
3	REMP	radiological environmental monitoring program
4	RN	service water
5	RPV	reactor pressure vessel
6	RV	reactor vessel
7		
8	SAMA	severe accident mitigation alternative
9	SAMDA	severe accident mitigation design alternatives
10	SBO	station blackout
11	SAR	Safety Analysis Report
12	SC	State species of concern
13	SEIS	Supplemental Environmental Impact Statement
14	SER	Safety Evaluation Report
15	SHPO	State Historical Preservation Officer
16	SR	significantly rare
17	SR	state route
18	SGTR	steam generator tube rupture
19	SS	safe shutdown
20	SSF	standby shutdown facility
21	Sv	sieverts
22		
23	T	threatened
24	TBq	terabecquerel
25		
26	UFSAR	Updated Final Safety Analysis Report
27	U_{RP}	long term replacement power cost
28	U.S.	United States
29		
30	yr	year

1.0 Introduction

Under the Nuclear Regulatory Commission's (NRC's) environmental protection regulations in Title 10 of the Code of Federal Regulations (CFR) Part 51, which implement the National Environmental Policy Act (NEPA), renewal of a nuclear power plant operating license (OL) requires the preparation of an environmental impact statement (EIS). In preparing the EIS, the NRC staff is required first to issue the statement in draft form for public comment and then issue a final statement after considering public comments on the draft. To support the preparation of the EIS, the staff has prepared a *Generic Environmental Impact Statement for License Renewal of Nuclear Plants* (GEIS), NUREG-1437, Volumes 1 and 2 (NRC 1996, 1999).^(a) The GEIS is intended to (1) provide an understanding of the types and severity of environmental impacts that may occur as a result of license renewal of nuclear power plants under 10 CFR Part 54, (2) identify and assess the impacts that are expected to be generic to license renewal, and (3) support 10 CFR Part 51 to define the number and scope of issues that need to be addressed by the applicants in plant-by-plant renewal proceedings. Use of the GEIS guides the preparation of complete plant-specific information in support of the OL renewal process.

Duke Energy Corporation (Duke)^(b) operates McGuire Nuclear Station, Units 1 and 2 (McGuire) in southwestern North Carolina under OLs NPF-9 and NPF-17, which were issued by the NRC. These OLs will expire in June 2021 for Unit 1 and in March 2023 for Unit 2. On June 13, 2001, Duke submitted an application to the NRC to renew the McGuire OLs for an additional 20 years under 10 CFR Part 54 (Duke 2001b). The application also included renewal for Catawba Nuclear Station in Rock Hill, South Carolina. A separate environmental evaluation is being conducted for Catawba Nuclear Station. Duke is a *licensee* for the purposes of its current OLs and an *applicant* for the renewal of the OLs. Pursuant to 10 CFR 54.23 and 51.53(c), Duke submitted an Environmental Report (ER) (Duke 2001a) in which Duke analyzed the environmental impacts associated with the proposed license renewal action, considered alternatives to the proposed action, and evaluated mitigation measures for reducing adverse environmental effects.

This report is the draft plant-specific supplement to the GEIS (the supplemental EIS [SEIS]) for the McGuire license renewal application. This SEIS is a supplement to the GEIS because it relies, in part, on the findings of the GEIS. The staff will also prepare a separate safety evaluation report in accordance with 10 CFR Part 54.

(a) The GEIS was originally issued in 1996. Addendum 1 to the GEIS was issued in 1999. Hereafter, all references to the "GEIS" include the GEIS and its Addendum 1.

(b) Duke Energy Corporation has held the license for McGuire Units 1 and 2 since September 16, 1997. Before this date, Duke Power Company held the license. Duke Power Company remains a division of Duke Energy Corporation.

1.1 Report Contents

The following sections of this introduction (1) describe the background for the preparation of this SEIS, including the development of the GEIS and the process used by the staff to assess the environmental impacts associated with license renewal; (2) describe the proposed Federal action to renew the OLs for McGuire; (3) discuss the purpose and need for the proposed action; and (4) present the status of Duke's compliance with environmental quality standards and requirements that have been imposed by Federal, State, regional, and local agencies that are responsible for environmental protection.

The ensuing chapters of this SEIS closely parallel the contents and organization of the GEIS. Chapter 2 describes the site, power plant, and interactions of the plant with the environment. Chapters 3 and 4, respectively, discuss the potential environmental impacts of plant refurbishment and plant operation during the renewal term. Chapter 5 contains an evaluation of potential environmental impacts of plant accidents and includes consideration of severe accident mitigation alternatives. Chapter 6 discusses the uranium fuel cycle and solid waste management, Chapter 7 discusses decommissioning, and Chapter 8 discusses alternatives to license renewal. Finally, Chapter 9 summarizes the findings of the preceding chapters and draws conclusions about the adverse impacts that cannot be avoided (the relationship between short-term uses of man's environment and the maintenance and enhancement of long-term productivity, and the irreversible or irretrievable commitment of resources). Chapter 9 also presents the staff's preliminary recommendation with respect to the proposed license renewal action.

Additional information is included in appendixes. Appendix A contains public comments received on the environmental review for license renewal and staff responses to those comments. Appendixes B through F, respectively, list the following:

- the preparers of the supplement
- the chronology of correspondence between NRC and Duke with regard to this SEIS
- the organizations contacted during the development of this SEIS
- Duke's compliance status in Table E-1
- GEIS environmental issues that are not applicable to McGuire.

1.2 Background

Use of the GEIS, which examines the possible environmental impacts that could occur as a result of renewing individual nuclear power plant OLS under 10 CFR Part 54, and the established license renewal evaluation process supports the thorough evaluation of the impacts of renewal of the OLS.

1.2.1 Generic Environmental Impact Statement

The NRC initiated a generic assessment of the environmental impacts associated with the license renewal term to improve the efficiency of the license renewal process by documenting the assessment results and codifying the results in the Commission's regulations. This assessment is provided in the GEIS, which serves as the principal reference for all nuclear power plant license renewal EISs.

In the GEIS, the staff documents the results of the systematic approach that was taken to evaluate the environmental consequences of renewing the licenses of individual nuclear power plants and operating them for an additional 20 years. For each potential environmental issue in the GEIS, the staff (1) describes the activity that affects the environment, (2) identifies the population or resource that is affected, (3) assesses the nature and magnitude of the impact on the affected population or resource, (4) characterizes the significance of the effect for both beneficial and adverse effects, (5) determines whether the results of the analysis apply to all plants, and (6) considers whether additional mitigation measures would be warranted for impacts that would have the same significance level for all plants.

The NRC's standard of significance was established using the Council on Environmental Quality (CEQ) terminology for "significantly" (40 CFR 1508.27, which requires consideration of both "context" and "intensity"). Using the CEQ terminology, the NRC established three significance levels—SMALL, MODERATE, or LARGE. The definitions of the three significance levels are set forth in the footnotes to Table B-1 of 10 CFR Part 51, Subpart A, Appendix B, as follows:

SMALL – Environmental effects are not detectable or are so minor that they will neither destabilize nor noticeably alter any important attribute of the resource.

MODERATE – Environmental effects are sufficient to alter noticeably, but not to destabilize, important attributes of the resource.

LARGE – Environmental effects are clearly noticeable and are sufficient to destabilize important attributes of the resource.

Introduction

1 In the GEIS, the staff assigns a significance level to each environmental issue, assuming that
2 ongoing mitigation measures would continue.

3
4 In the GEIS, the staff includes a determination of whether the analysis of the environmental
5 issue could be applied to all plants and whether additional mitigation measures would be
6 warranted. Issues are then assigned a Category 1 or a Category 2 designation. As set forth in
7 the GEIS, **Category 1** issues are those that meet all of the following criteria:

- 8
9 (1) The environmental impacts associated with the issue have been determined to apply either
10 to all plants or, for some issues, to plants having a specific type of cooling system or other
11 specified plant or site characteristic.
12
13 (2) A single significance level (i.e., SMALL, MODERATE, or LARGE) has been assigned to the
14 impacts (except for collective offsite radiological impacts from the fuel cycle and from high-
15 level waste and spent fuel disposal).
16
17 (3) Mitigation of adverse impacts associated with the issue has been considered in the analysis,
18 and it has been determined that additional plant-specific mitigation measures are likely to
19 not be sufficiently beneficial to warrant implementation.

20
21 For issues that meet the three Category 1 criteria, no additional plant-specific analysis is
22 required in the SEIS unless new and significant information is identified.

23
24 **Category 2** issues are those that do not meet one or more of the criteria of Category 1, and
25 therefore, additional plant-specific review for these issues is required.

26
27 In the GEIS, the staff assessed 92 environmental issues and determined that 69 qualified as
28 Category 1 issues, 21 qualified as Category 2 issues, and 2 issues were not categorized. The
29 latter 2 issues, environmental justice and chronic effects of electromagnetic fields, are to be
30 addressed in a plant-specific analysis. Of the 92 issues, 11 are related only to refurbishment, 6
31 are related only to decommissioning, 67 apply only to operation during the renewal term, and 8
32 apply to both refurbishment and operation during the renewal term. A summary of the findings
33 for all 92 issues in the GEIS is codified in Table B-1 of 10 CFR Part 51, Subpart A, Appendix B.

34 35 1.2.2 License Renewal Evaluation Process

36
37 An applicant seeking to renew its OLs is required to submit an ER as part of its application.
38 The license renewal evaluation process involves careful review of the applicant's ER and
39 assurance

that all new and potentially significant information not already addressed in or available during the GEIS evaluation is identified, reviewed, and assessed to verify the environmental impacts of the proposed license renewal.

In accordance with 10 CFR 51.53(c)(2) and (3), the ER submitted by the applicant must

- provide an analysis of the Category 2 issues in Table B-1 of 10 CFR Part 51, Subpart A, Appendix B, in accordance with 10 CFR 51.53(c)(3)(ii).
- discuss actions to mitigate any adverse impacts associated with the proposed action and environmental impacts of alternatives to the proposed action.

In accordance with 10 CFR 51.53(c)(2), the ER does not need to

- consider the economic benefits and costs of the proposed action and alternatives to the proposed action except insofar as such benefits and costs are either (1) essential for making a determination regarding the inclusion of an alternative in the range of alternatives considered or (2) relevant to mitigation.
- consider the need for power and other issues not related to the environmental effects of the proposed action and the alternatives.
- discuss any aspect of the storage of spent fuel within the scope of the generic determination in 10 CFR 51.23(a) in accordance with 10 CFR 51.23(b).
- contain an analysis of any Category 1 issue unless there is significant new information on a specific issue—this is pursuant to 10 CFR 51.53(c)(3)(iii) and (iv).

New and significant information is (1) information that identifies a significant environmental issue not covered in the GEIS and codified in Table B-1 of 10 CFR Part 51, Subpart A, Appendix B, or (2) information that was not considered in the analyses summarized in the GEIS and that leads to an impact finding that is different from the finding presented in the GEIS and codified in 10 CFR Part 51.

In preparing to submit its application to renew the McGuire OLS, Duke developed a process to ensure that information not addressed in or available during the GEIS evaluation regarding the environmental impacts of license renewal for McGuire would be properly reviewed before submitting the ER and to ensure that such new and potentially significant information related to renewal of the licenses for McGuire would be identified, reviewed, and assessed during the period of NRC review. Duke reviewed the Category 1 issues that appear in Table B-1 of 10

Introduction

CFR Part 51, Subpart A, Appendix B, to verify that the conclusions of the GEIS remained valid with respect to McGuire. This review was performed by personnel from Duke in consultation with Federal (other than NRC), State, and local environmental and natural resource agencies.

The NRC staff also has a process for identifying new and significant information. That process is described in detail in *Standard Review Plans for Environmental Reviews for Nuclear Power Plants, Supplement 1: Operating License Renewal* (ESRP), NUREG-1555, Supplement 1 (NRC 2000). The search for new information includes (1) review of an applicant's ER and the process for discovering and evaluating the significance of new information; (2) review of records of public comments; (3) review of environmental quality standards and regulations; (4) coordination with Federal, State, and local environmental protection and resource agencies; and (5) review of the technical literature. New information discovered by the staff is evaluated for significance using the criteria set forth in the GEIS. For Category 1 issues where new and significant information is identified, reconsideration of the conclusions for those issues is limited in scope to the assessment of the relevant new and significant information; the scope of the assessment does not include other facets of the issue that are not affected by the new information.

Chapters 3 through 7 discuss the environmental issues considered in the GEIS that are applicable to McGuire. At the beginning of the discussion of each set of issues, there is a table that identifies the issues to be addressed and lists the sections in the GEIS where the issue is discussed. Category 1 and Category 2 issues are listed in separate tables. For Category 1 issues for which there is no new and significant information, the table is followed by a set of short paragraphs that state the GEIS conclusion codified in Table B-1 of 10 CFR Part 51, Subpart A, Appendix B, followed by the staff's analysis and conclusion. For Category 2 issues, in addition to the list of GEIS sections where the issue is discussed, the tables list the subparagraph of 10 CFR 51.53(c)(3)(ii) that describes the analysis required and the draft SEIS sections where the analysis is presented. The draft SEIS sections that discuss the Category 2 issues are presented immediately following the table.

The NRC prepares an independent analysis of the environmental impacts of license renewal and compares these impacts with the environmental impacts of alternatives. The evaluation of the Duke license renewal application began with publication of a notice of acceptance for docketing and opportunity for a hearing in the *Federal Register* (FR; 66 FR 42893 [NRC 2001a]) on August 15, 2001. The staff published a notice of intent to prepare an EIS and conduct scoping (66 FR 44386 [NRC 2001b]) on August 23, 2001. Two public scoping meetings were held on September 25, 2001, in Huntersville, North Carolina. Comments received during the scoping periods were summarized in the *Environmental Impact Statement Scoping Process: Summary Report – McGuire Units 1 and 2, Huntersville, North Carolina*

(NRC 2002). Comments that are applicable to this environmental review are presented in Part 1 of Appendix A.

The staff followed the review guidance contained in NUREG-1555, Supplement 1, in the *Standard Review Plans for Environmental Reviews for Nuclear Power Plants, Supplement 1: Operating License Renewal* (NRC 2000). The staff and its contractors retained to assist the staff visited the McGuire site on September 24, 2001, to gather information and to become familiar with the site and its environs. The staff also reviewed the comments received during scoping and consulted with Federal, State, regional, and local agencies. A list of the organizations consulted is provided in Appendix D. Other documents related to McGuire were reviewed and are referenced.

This draft SEIS presents the staff's analysis that considers and weighs the environmental effects of the proposed renewal of the OLs for McGuire, the environmental impacts of alternatives to license renewal, and mitigation measures available for avoiding adverse environmental effects. Chapter 9, Summary and Conclusions, provides the NRC staff's preliminary recommendation to the Commission on whether or not the adverse environmental impacts of license renewal are so great that preserving the option of license renewal for energy-planning decisionmakers would be unreasonable.

A 75-day comment period will begin on the date of publication of the U.S. Environmental Protection Agency Notice of Filing of the draft SEIS, to allow members of the public to comment on the preliminary results of the NRC staff's review. During this comment period, two public meetings will be held in the vicinity of McGuire in June 2002. During these meetings, the staff will describe the preliminary results of the NRC environmental review and answer questions related to it to provide members of the public with information to assist them in formulating their comments.

1.3 The Proposed Federal Action

The proposed Federal action is renewal of the OLs for McGuire. The McGuire Nuclear Station is located in southwestern North Carolina, in northwestern Mecklenburg County on the shore of Lake Norman, approximately 27 km (17 mi) north-northwest of Charlotte and 10 km (6 mi) west of Huntersville. The plant has two Westinghouse-designed, pressurized, light-water reactors, each with a design rating for a net electrical power output of approximately 1129 megawatts electric (MW[e]). Water for the plant's once-through cooling system is drawn from and discharged back into Lake Norman. McGuire produces electricity to supply the needs of more than 619,000 homes. The current OL for Unit 1 expires on June 12, 2021, and that for Unit 2 on March 3, 2023. By letter dated June 13, 2001, Duke submitted an application to the NRC (Duke 2001b) to renew these OLs for up to an additional 20 years of operation.

1.4 The Purpose and Need for the Proposed Action

Although a licensee must have a renewed license to operate a reactor beyond the term of the existing OL, the possession of that license is just one of a number of conditions that must be met for the licensee to continue plant operation during the term of the renewed license. Once an OL is renewed, State regulatory agencies and the owners of the plant will ultimately decide whether the plant will continue to operate based on factors such as the need for power or other matters within the State's jurisdiction or the purview of the owners.

Thus, for license renewal reviews, the NRC has adopted the following definition of purpose and need (NRC 1996, Section 1.3):

The purpose and need for the proposed action (renewal of an operating license) is to provide an option that allows for power generation capability beyond the term of a current nuclear power plant operating license to meet future system generating needs, as such needs may be determined by State, utility, and where authorized, Federal (other than NRC) decisionmakers.

This definition of purpose and need reflects the Commission's recognition that, unless there are findings in the safety review required by the Atomic Energy Act or findings in the NEPA environmental analysis that would lead the NRC to reject a license renewal application, the NRC does not have a role in the energy-planning decisions of State regulators and utility officials as to whether a particular nuclear power plant should continue to operate. From the perspective of the licensee and the State regulatory authority, the purpose of renewing an OL is to maintain the availability of the nuclear plant to meet system energy requirements beyond the current term of the plant's license.

1.5 Compliance and Consultations

Duke is required to hold certain Federal, State, and local environmental permits, as well as meet relevant Federal and State statutory requirements. In the McGuire ER, Duke provided a list of the authorizations from Federal, State, and local authorities for current operations as well as environmental approvals and consultations associated with license renewal of McGuire. Authorizations and consultations most relevant to the proposed OL renewal action are summarized in Table 1-1. The full list of authorizations and consultations provided by Duke is included in Appendix E.

Table 1-1. Federal, State, and Local Authorizations and Consultations

Agency	Authority	Requirement	Number	Permit Expiration or Consultation Date	Activity Covered
NRC	Atomic Energy Act, 10 CFR Part 50	Operating license	NPF-9 (Unit 1) NPF-17 (Unit 2)	June 12, 2021 (Unit 1) March 3, 2023 (Unit 2)	Operation of McGuire Units 1 and 2
FWS	Endangered Species Act, Section 7	Consultation	NA	Consultation initiated October 10, 2001	Operation during renewal term
NCDENR	Clean Water Act, Section 402	NPDES wastewater permit	NCOO24392	February 28, 2005	Permit for discharge of wastewater and once-through cooling water to discharge canal that empties into Lake Norman
NCDENR	Clean Water Act, Section 402	NPDES stormwater permit	NCS000020	February 28, 2005	Collection, treatment, and discharge of stormwater
Mecklenburg County Department of Environmental Protection	Clean Air Act, Section 112	Air emissions permit	00-019-269	Renewed annually	Emissions from diesel emergency generators, miscellaneous diesel engines, and other miscellaneous units
NCDCR	National Historic Preservation Act, Section 106	Consultation	NA	Consultation initiated January 31, 2000	Impact on sites listed or eligible for listing in the National Register of Historic Places
FWS - U.S. Fish and Wildlife Service.					
NCDCR - North Carolina Department of Cultural Resources.					
NCDENR - North Carolina Department of Environment and Natural Resources.					
NPDES - National Pollutant Discharge Elimination System.					
NA - Not applicable.					

Introduction

The staff has reviewed the list and consulted with the appropriate Federal, State, and local agencies to identify any compliance or permit issues or significant environmental issues of concern to the reviewing agencies. These agencies did not identify any new and significant environmental issues. The McGuire ER states that Duke is in compliance with applicable environmental standards and requirements for McGuire. The staff has also not identified any environmental issues that are both new and significant.

1.6 References

10 CFR 51. Code of Federal Regulations, Title 10, *Energy*, Part 51, "Environmental Protection Regulations for Domestic Licensing and Related Regulatory Functions."

10 CFR 54. Code of Federal Regulations, Title 10, *Energy*, Part 54, "Requirements for Renewal of Operating Licenses for Nuclear Power Plants."

40 CFR 1508. Code of Federal Regulations, Title 40, *Protection of Environment*, Part 1508, "Terminology and Index."

Atomic Energy Act of 1954 (AEA). 42 USC 2011, et seq.

Clean Air Act (CAA). 42 USC 7401, et seq.

Duke Energy Corporation (Duke). 2001a. *Applicant's Environmental Report – Operating License Renewal Stage — McGuire Nuclear Station*. Charlotte, North Carolina.

Duke Energy Corporation (Duke). 2001b. *Application to Renew the Operating Licenses of McGuire Nuclear Station, Units 1 and 2 and the Catawba Nuclear Station, Units 1 and 2*. Charlotte, North Carolina.

Endangered Species Act (ESA). 16 USC 1531, et seq.

Federal Water Pollution Control Act. 33 USC 1251, et seq. (Also known as the Clean Water Act [CWA]).

National Environmental Policy Act of 1969 (NEPA). 42 USC 4321, et seq.

National Historic Preservation Act of 1966 (NHPA). 16 USC 470, et seq.

1 U.S. Nuclear Regulatory Commission (NRC). 1996. *Generic Environmental Impact Statement*
2 *for License Renewal of Nuclear Plants*. NUREG-1437, Volumes 1 and 2, Washington, D.C.

3
4 U.S. Nuclear Regulatory Commission (NRC). 1999. *Generic Environmental Impact Statement*
5 *for License Renewal of Nuclear Plants, Main Report*, "Section 6.3 - Transportation, Table 9.1
6 Summary of findings on NEPA issues for license renewal of nuclear power plants, Final
7 Report." NUREG-1437, Volume 1, Addendum 1, Washington, D.C.

8
9 U.S. Nuclear Regulatory Commission (NRC). 2000. *Standard Review Plans for Environmental*
10 *Reviews for Nuclear Power Plants, Supplement 1: Operating License Renewal*. NUREG-1555,
11 Supplement 1, Washington, D.C.

12
13 U.S. Nuclear Regulatory Commission (NRC). 2001a. "Duke Energy Corporation, McGuire,
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2.0 Description of Nuclear Power Plant and Site and Plant Interaction with the Environment

The Duke Energy Corporation's (Duke's) McGuire Nuclear Station, Units 1 and 2 (McGuire) is located on the shore of Lake Norman in North Carolina's Mecklenburg County approximately 27 km (17 mi) north-northwest of Charlotte, North Carolina. The plant consists of two units (Units 1 and 2) that are the subject of this action. Each unit is a pressurized light-water reactor (LWR) with four steam generators producing steam that turns turbines to generate electricity. Lake Norman is used as the sources of cooling and process water for McGuire. The plant and its environs are described in Section 2.1, and the plant's interaction with the environment is presented in Section 2.2.

2.1 Plant and Site Description and Proposed Plant Operation During the Renewal Term

McGuire is located on 234 ha (577 ac) of Duke-owned land in southwestern North Carolina. Figures 2-1 and 2-2 show the site location and features within 80 km and 10 km (50 mi and 6 mi), respectively. The site is surrounded by an exclusion area whose radius measures 0.76 km (0.47 mi) and covers 182.4 ha (450.5 ac) (Duke 2001a).

The McGuire site is bounded to the west by the Catawba River and to the north by Lake Norman. Lake Norman is a 13,156-ha (32,510-ac) lake that was formed by the impoundment of the Catawba River by the Cowan's Ford Dam hydroelectric station (owned and operated by Duke Power). Lake Norman achieved full pond level in 1964. Cowan's Ford Dam is immediately west of the site and on the Catawba River channel.

The region surrounding McGuire is considered to have a high population density based on the definitions applied to case study sites in the *Generic Environmental Impact Statement for License Renewal of Nuclear Plants* (GEIS), NUREG-1437, Volumes 1 and 2 (NRC 1996, 1999b).^(a) The area around McGuire is experiencing a rapid change from a rural to a suburban environment (Duke 2001a). Huntersville (population 25,000), North Carolina, is the nearest town (Duke 2001a). The town center is located approximately 10 km (6 mi) to the east of the plant.

(a) The GEIS was originally issued in 1996. Addendum 1 to the GEIS was issued in 1999. Hereafter, all references to the "GEIS" include the GEIS and its Addendum 1.

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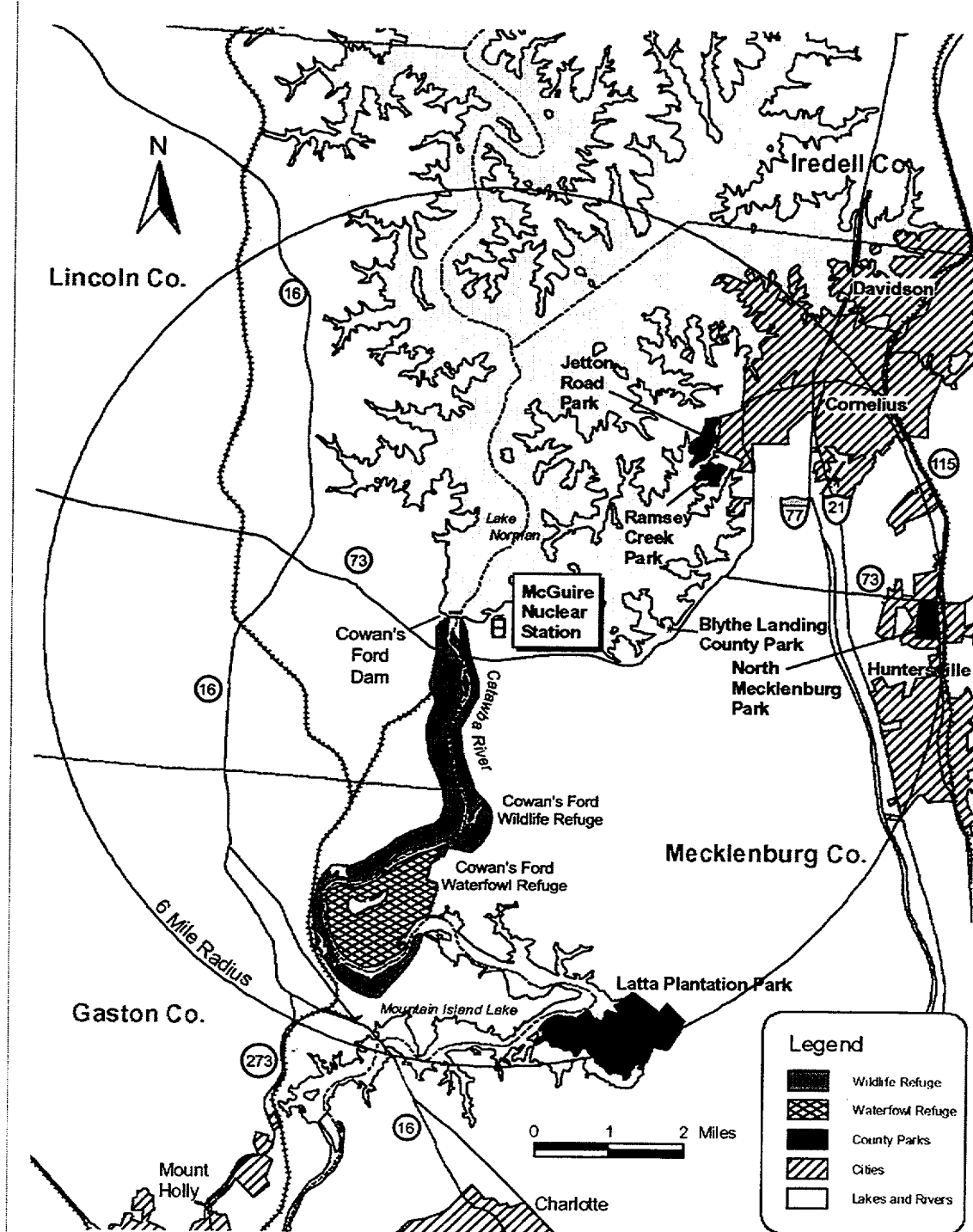


Figure 2-2. Location of McGuire Nuclear Station, Units 1 and 2, 10-km (6-mi) Region

Plant and the Environment

1 The McGuire site has approximately 1345 full-time workers employed by Duke and site
2 contractors during normal plant operations. Duke refuels each reactor unit at McGuire on an
3 18- to 24-month schedule, when site employment increases by as many as 1015 workers for
4 temporary duty (30 to 40 days).

5
6 The McGuire exclusion area varies in elevation from 198 m to 244 m (650 to 800 ft) (Duke
7 2001a), and its topography is rolling (NRC 1996). The exclusion area is dominated by Cecil
8 sandy loam and harbors typical piedmont plant communities and cover types, predominantly
9 hardwood-pine forests and marshes and wetlands (Duke 2001a). The majority of land in the
10 area immediately around McGuire is forested, pasture, cropland, or residential developments,
11 each contributing significant proportions to the total land use. The shoreline of Lake Norman is
12 developed with vacation and permanent residences, campgrounds, boat launches, marinas,
13 and golf courses. The 270-ha (668-ac) Cowan's Ford Wildlife Refuge (owned and operated by
14 Mecklenburg County Parks and Recreation Department) and the Cowan's Ford Waterfowl
15 Refuge (managed by the North Carolina Wildlife Resources Commission) are located just south
16 of the McGuire exclusion area along the shores of Mountain Island Lake. These areas, as well
17 as adjacent lands, have been officially designated as Important Bird Areas (IBAs) by the
18 National Audubon Society because of their rich avian diversity (Duke 2001a).

19
20 Five parks (Blythe Landing County Park, Jetton Road Park, Latta Plantation Park, North
21 Mecklenburg Park, and Ramsey Park), located in and owned by Mecklenburg County, are
22 within a 10-km (6-mi) radius of the McGuire plant. Five state parks (Andrew Jackson State
23 Park, Crowders Mountain State Park, Lake Norman State Park, Morrow Mountain State Park,
24 and South Mountain State Park), Kings Mountain National Military Park, and the Catawba
25 Indian Reservation are located within 80 km (50 mi) of the McGuire plant (Duke 2001a).

26 27 **2.1.1 External Appearance and Setting**

28
29 Because of the large amount of timber adjacent to the site, the nuclear plant is visible from only
30 a few locations on the land. It is readily visible from adjacent locations along the lake shore.
31 The most obvious structures are the transmission lines that are visible from North Carolina
32 Highway 73 (NC-73), which runs along the southern edge of the site.

33
34 McGuire Units 1 and 2 each have a separate reactor building, turbine building, and switchyard.
35 The following buildings and features are common to both units: service building, auxiliary
36 building, intake structures (upper level and lower level), discharge structure and discharge
37 canal, standby nuclear service water pond, and independent spent fuel storage installation
38 (ISFSI) (Duke 2001a).

1 The ISFSI was added to McGuire to expand the storage capacity for spent fuel. The initial
2 loading of spent fuel into the dry storage facility took place in 2001. The storage of spent fuel in
3 the ISFSI is conducted under a general permit issued in accordance with 10 CFR 72.210. The
4 ISFSI is outside the scope of this review.

5
6 The McGuire site lies near the center of the Piedmont physiographic province. The Piedmont
7 is characterized by rolling hills and numerous small streams and rivers. It is a northeast-
8 trending zone from Georgia through Virginia that varies in width from about 130 to 190 km (80
9 to 120 mi) (Duke 2001a). The Fall Line, which divides the Piedmont from the Coastal Plain
10 physiographic province to the southeast, lies 100 km (65 mi) from the site.

11
12 The Piedmont province is underlain by five narrow northeast-trending belts of metamorphosed
13 sedimentary rock. The McGuire site is within the Charlotte Belt. These rocks, originally formed
14 during the lower Paleozoic, are now in the form of mica schist and gabbro. Although there are
15 numerous faults in the Piedmont region, there are no identifiable faults or other geological
16 structures that could be expected to localize earthquakes in the immediate vicinity of the
17 McGuire site (NRC 1976).

18 19 **2.1.2 Reactor Systems**

20
21 The McGuire site is shown in Figure 2-3. Each unit is a pressurized LWR with four steam
22 generators that produce steam that turns turbines to generate electricity. Each unit, designed
23 and fabricated by the Westinghouse Electric Corporation, is designed to operate at core power
24 levels up to 3411 megawatts thermal (MW[t]), with a corresponding net electrical output of
25 approximately 1129 MW(e) (Duke 2001a).

26
27 The nuclear steam supply system for each unit is housed in a separate free-standing steel
28 containment structure within a reinforced concrete shield building. The containment employs
29 the ice condenser pressure-suppression concept. The containment is designed to withstand
30 environmental effects and the internal pressure and temperature accompanying a postulated
31 loss-of-coolant accident or steam-line break. Together with its engineered safety features, the
32 containment structure for each unit is designed to adequately retain fission products that
33 escape from the reactor coolant system.

34
35 McGuire is licensed for fuel that is slightly enriched uranium dioxide, up to 4.75 percent by
36 weight uranium-235 (Duke 2001a). McGuire has several different fuel designs that are used for
37 the production of electricity. The Mark-BW design has a maximum fuel assembly burnup of
38 55,000 megawatt days/metric tons of uranium (MWd/MTU) and a maximum licensed fuel pin
39

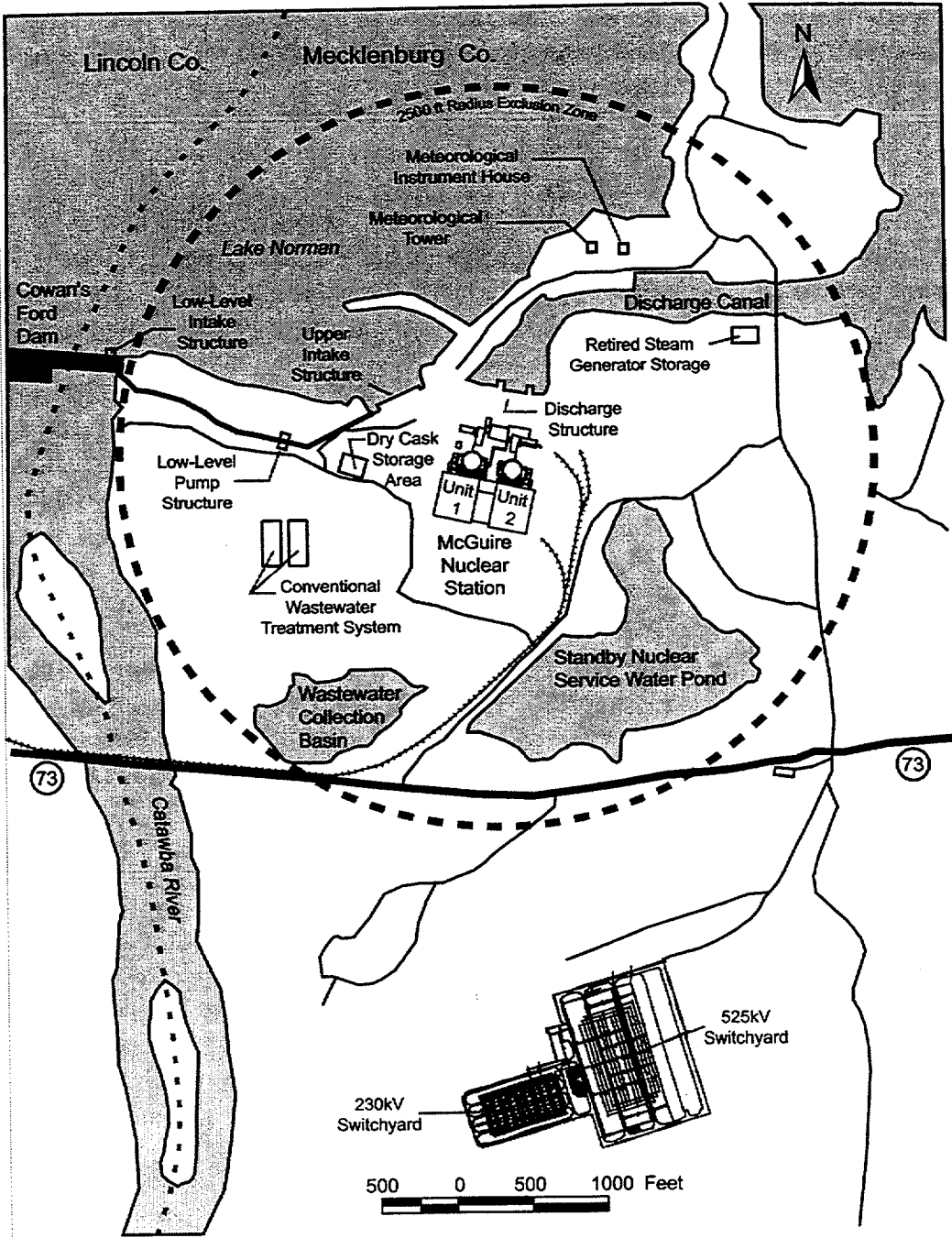


Figure 2-3. McGuire Nuclear Station

1 burnup of 60,000 MWd/MTU. The Westinghouse Robust Fuel Assembly design does not have
2 a maximum fuel assembly burnup limit; however, this burnup value would be limited by the
3 maximum licensed fuel pin burnup limit of 60,000 MWd/MTU (Duke 2001a).

4 5 **2.1.3 Cooling and Auxiliary Water Systems**

6
7 The site currently uses water from Lake Norman for main condenser cooling and process
8 water. Water is withdrawn from the lake at an average daily rate (two-unit operation) of about
9 111 m³/s (2530 million gpd), circulated through the two units and discharged back into the lake
10 through the discharge canal. The plant has an upper-level intake and a separate, lower-level
11 intake structure.

12
13 For most of the year, cooling and process water is withdrawn from Lake Norman through the
14 upper-level intake structure. The upper-level intake structure is located at the lake surface at
15 the end of the intake channel. It withdraws from the surface water layers of the lake
16 (epilimnion). The water in the intake channel flows through trash bars and through 1-cm (3/8
17 in.) mesh vertical traveling screens before entering the McGuire plant. Water velocity in the
18 upper intake channel is less than 0.3 m/s (1 ft/s).

19
20 During periods of high lake-surface temperature, cooler water (hypolimnion layer) is withdrawn
21 from the lake bottom through the lower-level intake structure. The lower-level intake structure
22 is located west of the upper intake structure and approximately 30 m (100 ft) below the lake
23 surface. Water from the lower intake structure is pumped by a pumping station up to a canal
24 that discharges the cooler water in front of the upper intake structure. The water from the lower
25 intake structure supplements, but cannot completely replace, the surface water flow from the
26 upper intake channel. Thus, water from the lower intake structure drawn primarily during the
27 hot summer months reduces the temperature of the water that is drawn into the plant for
28 cooling. This results in a lower station discharge water temperature. There are no traveling
29 screens on the lower-level intake structure. Water velocity through the lower-level intake
30 structure, when operating, can be as high as 0.43 m/s (1.4 ft/s).

31
32 Operation of the rotating vertical traveling screens can be in either an automatic or manual
33 mode. Automatic rotation of the screens is controlled by differential pressure across the screen
34 surface. Manual operation and cleaning of the traveling screens is prescribed weekly.
35 Backwash water and screen debris are discharged into a refuse removal trench, which drains
36 into a debris retention basket.

37
38 The increase in temperature of cooling system water discharged back into Lake Norman is
39 related to flow and intake water temperature. During the winter, when the incoming water is the

1 coolest and the flow is the lowest, the increase in temperature is 13.7°C (24.7°F). During the
2 summer, when the intake temperatures are the warmest and the flow is the highest, the
3 temperature increase is 8.6°C (15.5°F).

4
5 Potable water at McGuire is supplied by the Charlotte-Mecklenburg Utilities Department
6 (CMUD) water supply system. Six groundwater wells provide specific low-volume uses (e.g.,
7 irrigation, remote restrooms) with a combined maximum pumping rate of 4.3 L/s (68 gpm).

8 9 **2.1.4 Radioactive Waste Management Systems and Effluent Control Systems**

10
11 McGuire uses liquid, gaseous, and solid radioactive waste management systems to collect and
12 process the liquid, gaseous, and solid wastes that are the by-products of McGuire operation.
13 These systems process radioactive liquid, gaseous, and solid effluents before they are released
14 to the environment. The waste disposal systems for McGuire meet the design objectives of
15 10 CFR Part 50, Appendix I (Numerical Guides for Design Objectives and Limiting Conditions
16 for Operations to Meet the Criterion "As Low As Reasonably Achievable" for Radioactive
17 Material in Light-Water Cooled Nuclear Power Reactor Effluents), and control the processing,
18 disposal, and release of radioactive liquid, gaseous, and solid wastes. Radioactive material in
19 the reactor coolant is the source of gaseous, liquid, and solid radioactive wastes in LWRs.
20 Radioactive fission products build up within the fuel as a consequence of the fission process.
21 These fission products are contained in the sealed fuel rods, but small quantities escape from
22 the fuel rods and contaminate the reactor coolant. Neutron activation of the primary coolant
23 system also is responsible for coolant contamination.

24
25 Nonfuel solid wastes result from treating and separating radionuclides from gases and liquids
26 and from removing contaminated material from various reactor areas. Solid wastes also consist
27 of reactor components, equipment, and tools removed from service, as well as contaminated
28 protective clothing, paper, rags, and other trash generated from plant design modifications and
29 operations and routine maintenance activities. Solid wastes are shipped to a waste processor
30 for volume reduction before disposal at a licensed burial site (Duke 2001a). Spent resins and
31 filters are stored or packaged for shipment to a licensed offsite processing or disposal facility
32 (Duke 2001a).

33
34 Fuel rods that have exhausted a certain percentage of their fuel and are removed from the
35 reactor core for disposal are called spent fuel. Each unit is refueled approximately every 18 to
36 24 months. Refueling outages are staggered so both units are not in an outage at the same
37 time (Duke 2001a). Spent fuel is stored onsite in one of the two spent fuel pools or in
38 containers in the McGuire ISFSI (Duke 2001a). Each unit has its own spent fuel pool located in
39 the auxiliary building. Spent fuel storage in the McGuire ISFSI was initiated in 2001.
40

1 The waste disposal system used for processing liquid, gaseous, and solid wastes is common to
2 Units 1 and 2, with the exception of the reactor coolant drain tanks located in each reactor
3 containment (Duke 2000a).

4
5 The offsite dose calculation manual (ODCM) for McGuire (Duke 2001e) describes the methods
6 used for calculating radioactivity concentrations in the environment and the estimated potential
7 offsite doses associated with liquid and gaseous effluents from McGuire. The ODCM also
8 specifies controls for release of liquid and gaseous effluents to ensure compliance with the
9 following:

- 10
11 • The concentration of radioactive liquid effluents released from the site to the
12 unrestricted area will not exceed 10 times the concentration specified in 10 CFR Part 20,
13 Appendix B, Table 2, Column 2, for radionuclides other than dissolved or entrained
14 gases. For dissolved or entrained noble gases, the concentration shall not exceed
15 7.4 Bq/mL (0.0002 μ Ci/mL).
- 16
17 • The dose or dose commitment per reactor to a member of the public from any radio-
18 active materials in liquid effluents released to unrestricted areas shall be limited to the
19 design objectives of 10 CFR Part 50, Appendix I; (1) less than or equal to 0.015 mSv
20 [1.5 mrem] to the total body and less than or equal to 0.05 mSv [5 mrem] to any organ
21 during any calendar quarter, and (2) less than or equal to 0.03 mSv [3 mrem] to the total
22 body and less than or equal to 0.1 mSv [10 mrem] to any organ during any calendar
23 year).
- 24
25 • The dose rate due to radioactive materials released in gaseous effluents from the site to
26 areas at and beyond the site boundary shall be limited to (1) less than or equal to
27 5 mSv/yr (500 mrem/yr) to the total body and less than or equal to 30 mSv
28 (3000 mrem/yr) to the skin due to noble gases and (2) less than or equal to 15 mSv/yr
29 (1500 mrem/yr) to any organ due to iodine-131, iodine-133, tritium, and for all
30 radioactive materials in particulate form with half-lives greater than 8 days per NUREG-
31 1301 (NRC 1991).
- 32
33 • The air dose per reactor to areas at and beyond the site boundary due to noble gases
34 released in gaseous effluents shall be limited to the design objectives of 10 CFR Part
35 50, Appendix I (i.e., less than or equal to 0.1 mGy (10 mrad) for gamma radiation and
36 less than or equal to 0.2 mGy (20 mrad) for beta radiation during any calendar year).
- 37
38 • The dose to any individual member of the public from the nuclear facility operations will
39 not exceed the maximum limits of 40 CFR Part 190 (i.e., less than 0.25 mSv [25 mrem])
40 and 10 CFR Part 20 (i.e., less than or equal to 5 mSv [0.5 rem] in a year and less than
41 or equal to 0.02 mSv [2 mrem] in any hour).

2.1.4.1 Liquid Waste Processing Systems and Effluent Controls

All radioactive and potentially radioactive liquids generated in the plant are collected, segregated, and processed. Most reactor- or primary-grade liquids are recycled. Potentially contaminated radioactive liquid wastes in the plant are collected in tanks in the auxiliary building and processed by filtration, demineralization, or evaporation prior to their monitoring and discharge to Lake Norman (Duke 2001a). Liquid wastes from the auxiliary building floor drains, sumps, and equipment drains, as well as from the plant's containment sumps, laboratory drains, and waste evaporator feed tank drainage are collected in the floor drain tank (Duke 2000a). Dependent on the activity of liquid wastes in the floor drain tank, further processing (i.e., filtering, chemical treatment, demineralization) may be required prior to collection in one of two waste monitor tanks (Duke 2000a). Liquid wastes from the laundry hot shower tank also are collected in the waste monitor tanks after filtering (Duke 2000a). From the waste monitor tanks, liquid wastes are sampled and monitored. When they are found to be within the regulated levels, they then are discharged into the condenser cooling water system (i.e., condenser circulating water) that flows into Lake Norman (Duke 2000a). Condensate from the containment ventilation units is collected in the ventilation unit condensate drain tank (Duke 2000a). Liquid wastes from this tank are monitored and discharged into the condenser cooling water system (i.e., condenser circulating water) flowing into Lake Norman similar to the discharge from the waste monitor tanks.

Liquid wastes from the turbine building sump (typically not contaminated) are monitored and released to the conventional wastewater system and the wastewater collection basin discharge point to the Catawba River downstream of Cowan's Ford Dam (Duke 2001e). If monitoring shows elevated radioactivity levels in the Turbine Building sump, liquid waste is routed into the floor drain tank for processing as described above and eventual discharge to Lake Norman (Duke 2001e).

The ODCM prescribes the alarm/trip setpoints for the liquid effluent radiation monitors; the setpoints are derived from 10 times the effluent concentration limits provided in 10 CFR Part 20, Appendix B, Table 2, Column 2. Liquid effluent radiation monitors are located on the waste monitor tank release line, the containment ventilation unit condensate drain tank release lines, and the turbine building sump release line (Duke 2001e). The alarm/trip setpoint for each liquid effluent monitor is based on the measurements of radioactivity in a batch of liquid to be released or in the continuous liquid discharge (Duke 2001e).

During 2000, there were 246 batch releases of liquid effluents for the two units in a total volume of $1.37\text{E}+7$ L ($3.62\text{E}+6$ gal) prior to dilution (Duke 2001c). The combined liquid waste volume prior to dilution for batch and continuous releases for 2000 was $3.35\text{E}+8$ L ($8.84\text{E}+7$ gal) (Duke 2001c). The liquid waste holdup capacity for the plant is approximately $1.48\text{E}+6$ L.

(390,000 gal) (Duke 2001a). The actual liquid waste generated is reported in the *McGuire Nuclear Station Annual Radioactive Effluent Release Report* (Duke 2001c).

Duke does not anticipate any increase in liquid waste releases during the renewal period.

2.1.4.2 Gaseous Waste Processing Systems and Effluent Controls

The waste gas system is designed to remove fission gases from radioactive contaminated fluids and contain these gases. Fission gases are removed from other systems to the maximum extent possible and contained in the waste gas system. The system is designed so that storage and subsequent decay of these gases can eliminate, to a large extent, the need for regularly scheduled discharge of these radioactive gases from the system into the atmosphere during normal plant operation. There are times, however, when the release of radioactive gas may become necessary. As a result, there are provisions to sample and isolate each of the decay tanks.

The waste gas system, containment and auxiliary building ventilation, and flow from the condenser air ejectors exhaust into the unit vents (Duke 2001e). These four contributors to the unit vent exhaust are discussed below. The unit vents are the primary (major) gaseous release points from the plant (Duke 2001e).

- Waste Gas System. The waste gas system in the auxiliary building (Duke 2000a) is shared between the two reactor units and consists of two waste gas compressors, two catalytic hydrogen recombiners, six gas decay storage tanks for use during normal power generation, and two gas decay storage tanks for use during shutdown and startup operations (Duke 2001e). Letdown flow from the reactor coolant system is processed through the waste gas system, and the resultant gases (hydrogen, nitrogen, and small quantities of the fission products xenon and krypton) are collected in the waste gas decay storage tanks. Gases are allowed to decay in these tanks, then are released at permissible rates and activity to the Unit 1 vent as prescribed by the ODCM (Duke 2001e).
- Containment Ventilation. The containment ventilation includes atmosphere from the containment purge, containment air release and addition, and containment annulus (Duke 2000a). The containment atmosphere will pass through a charcoal adsorber and a high-efficiency particulate air (HEPA) filter prior to being exhausted into either the Unit 1 or Unit 2 vent (Duke 2001e).

- 1 • Auxiliary Building Ventilation. Radioactive gases generated within the auxiliary building
2 will be exhausted through the building's ventilation system. Exhausted air is monitored
3 and, upon radiation monitor alarm, the exhaust air is diverted through a charcoal
4 adsorber and a HEPA filter prior to being released to the Unit 1 or Unit 2 vent (Duke
5 2001e).
- 6 • Condenser Air Ejectors. Gases from the condenser air ejectors are monitored
7 continuously and discharged into either the Unit 1 or Unit 2 vent (Duke 2000a).

8
9
10 Secondary (minor) release points include the waste management facility, the waste handling
11 area, and the Unit 2 staging building (Duke 2001e). Exhausts from these three areas are
12 monitored continuously and, upon a high radiation alarm, the supply and exhaust ventilation
13 fans are stopped (Duke 2000a).

14
15 Radioactive gaseous wastes from McGuire are released primarily through the Unit 1 and 2
16 vents. The exhaust streams that flow into the unit vents (i.e., waste gas decay storage tanks,
17 containment ventilation, auxiliary building ventilation, and condenser air ejectors) are monitored
18 for radioactivity. The vents for each unit are continuously monitored for noble gases,
19 radioiodines, and particulate activity (Duke 2000a). The ODCM prescribes alarm/trip setpoints
20 for these effluent monitors and control instrumentation to ensure that the alarm/trip will occur
21 prior to exceeding the limits of 10 CFR Part 20 for gaseous effluents (Duke 2001e).

22
23 Duke does not anticipate any increase in gaseous releases during the renewal period.

24 25 **2.1.4.3 Solid Waste Processing**

26
27 Solid radioactive wastes from McGuire consist of spent resin, spent (contaminated) filter
28 elements, contaminated oils and sludges, and miscellaneous solid materials (Duke 2000a,
29 Duke 2001a). Spent resin is flushed from plant demineralizers into spent resin storage tanks.
30 The spent resin then is processed by dewatering or solidification and packaged in a cask liner,
31 which is placed in a shielded cask truck (Duke 2000a). Spent filter elements are removed from
32 their housing using filter-handling tools and filter transfer shields. They are transferred to a
33 shielded filter storage bunker in the waste drumming area (Duke 2000a). Contaminated oils
34 and sludges either are pumped to a processing area for solidification or are shipped to an
35 offsite vendor for processing (Duke 2001a). Miscellaneous solid materials include rubber
36 gloves, plastic bags, contaminated clothing, contaminated rags, and contaminated tools (Duke
37 2001a).

38
39 Lower-activity wastes (i.e., miscellaneous solid materials) are processed at an offsite waste
40 processing facility for volume reduction or segregation prior to disposal at a licensed facility

such as Barnwell, South Carolina, or Envirocare of Utah (Duke 2001a). Higher-activity wastes (i.e., spent resins) are typically sent directly to a licensed disposal facility such as Barnwell, South Carolina (Duke 2001a).

Disposal and transportation of solid wastes are performed in accordance with the applicable requirements of 10 CFR Part 61 and 10 CFR Part 71, respectively. There are no releases to the environment from radioactive solid wastes created at McGuire.

In 2000, McGuire Units 1 and 2 made eight shipments of solid waste with a volume of 47 m³ (1650 ft³) and a total activity of 0.19 TBq (5 Ci) (Duke 2001c). These shipments are representative of the shipments made in the past several years and are not expected to change appreciably during the license renewal period.

2.1.5 Nonradioactive Waste Systems

Nonradioactive solid wastes from McGuire are disposed of in the onsite landfill or in one of several offsite landfills operated by Mecklenburg County (Duke 2001a). The onsite landfill typically handles the following types of wastes: asbestos, empty containers and drums, insulation (nonasbestos), nonhazardous-spill cleanup, conventional wastewater sludge, alkaline batteries, and oil-contaminated materials. This landfill is permitted by the North Carolina Department of Environmental and Natural Resources (NCDENR), Solid Waste Section (Duke 2001a). General office trash is disposed in one of several offsite landfills operated by Mecklenburg County (Duke 2001a).

Nonradioactive liquid wastes are sampled and treated according to the site National Pollutant Discharge Elimination System (NPDES) permits issued to McGuire by the North Carolina Department of Environmental and Natural Resources (Duke 2001a). These wastes originate from system drainage/leakage, water treatment activities, housekeeping/cleaning wastes, stormwater runoff, and floor and yard drains (Duke 2001a). Sanitary wastes are treated offsite by the CMUD (Duke 2001a).

2.1.6 Plant Operation and Maintenance

Routine maintenance performed on plant systems and components is necessary for safe and reliable operation of a nuclear power plant. Maintenance activities conducted at McGuire include inspection, testing, and surveillance to maintain the current licensing basis of the plant and to ensure compliance with environmental and safety requirements. Certain activities can be performed while the reactor is operating. Others require that the plant be shut down. Long-term outages are scheduled for refueling and for certain types of repairs or maintenance, such as replacement of a major component. Duke refuels each of the McGuire units every 18 to 24 months (Duke 2001a). Each outage is typically scheduled to last approximately 30 to

40 days; the outage schedules are staggered so that both units are not in an outage at the same time (Duke 2001a). One-third of the core is offloaded at each refueling. Approximately 1015 additional workers are onsite during a typical outage (Duke 2001a).

Duke provided an appendix in *Duke Energy Company McGuire Nuclear Station Updated Final Safety Analysis Report* regarding the aging management review to manage the effects of aging on systems, structures, and components in accordance with 10 CFR Part 54 (Duke 2000a). Chapter 3 and Appendix B of the McGuire license renewal application specifies the programs and activities that will manage the effects of aging during the license renewal period (Duke 2001b). Duke expects to conduct the activities related to the management of aging effects during plant operation or normal refueling and other outages but plans no outages specifically for refurbishment activities. Duke has no plans to add additional full-time staff (nonoutage workers) at the plant during the period of the renewed licenses.

2.1.7 Power Transmission System

Two switchyards connect the McGuire plant transmission lines to the transmission system: a 230-kV switchyard for Unit 1 and a 525-kV switchyard for Unit 2. The switchyards are located south of Highway NC-73 (see Figure 2-4). Power from Unit 1 is transmitted to the 230-kV switchyard over two separate three-phase 230-kV transmission lines with an average length of 1.2 km (4000 ft) (Figure 2-4). Power from Unit 2 is transmitted to the 525-kV switching station over two separate three-phase 525-kV transmission lines with an average length of 1 km (3300 ft) (Figure 2-4). The 230- and 525-kV lines are designed to meet the heavy loading condition as defined in the National Electrical Safety Code, 7th Edition (Duke 2001). The 230-kV switching station is tied into the Duke 230-kV network by seven double-circuit overhead lines. The 525-kV switching station is east of the 230-kV switching station and is tied into the Duke 525-kV network by four single-circuit overhead lines.

The right-of-way for the 525-kV lines is 151.5 m (500 ft) wide. The right-of-way for the 230-kV lines is 60.6 m (200 ft) wide (Gaddy 2001). Duke has a well-established set of management practices for right-of-way maintenance. These best management practices include vegetation management; erosion and sediment control; soil stabilization; stream and wetland protection; and protection of sensitive areas and sensitive species. Vegetation management consists of mowing and herbicide application (Gaddy 2001). Arsenal and Accord with Garlon 4A or Krenite are used for stump treatments and basal applications. Each of these products has been evaluated for safety and environmental concerns. In particular, Arsenal and Accord are approved for use in wetland areas. Following initial treatment with Arsenal and Accord, rights-of-way are maintained thereafter on an approximate 3-year rotation. Subsequent herbicide applications are limited primarily to trees that could grow into transmission lines (Duke 2001a).

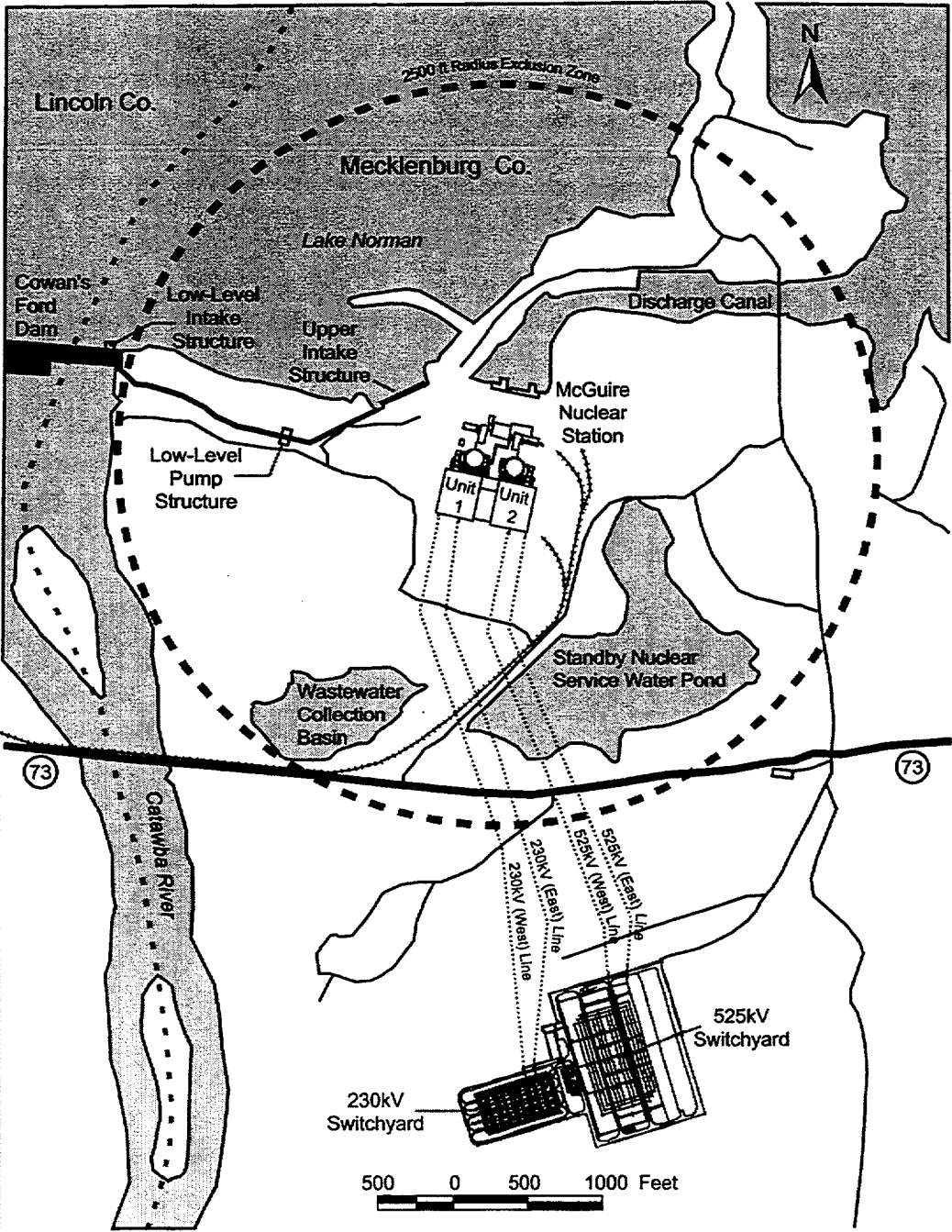


Figure 2-4. Transmission Lines Attributable to McGuire Nuclear Station

Duke maintains a working relationship with the North Carolina Department of Environment and Natural Resources (NCDENR) Natural Heritage Program and the U.S. Fish and Wildlife Service (FWS). Duke communicates with these agencies about pertinent natural heritage data such as Federal- and State-listed species, special habitats, and new findings. Information from the North Carolina Natural Heritage Program database is used to establish new and review existing vegetation management programs for the rights-of-way (Duke 2001a).

The transmission line connecting McGuire to the Oconee Nuclear Station was evaluated previously in the Supplemental Generic Environmental Impact Statement for license renewal of the Oconee Nuclear Station (NRC 1999a).

2.2 Plant Interaction with the Environment

Sections 2.2.1 through 2.2.8 provide general descriptions of the environment as background information. They also provide detailed descriptions where needed to support the analysis of potential environmental impacts of refurbishment and operation during the renewal term, as discussed in Chapters 3 and 4. Section 2.2.9 describes the historic and archaeological resources in the area, and Section 2.2.10 describes possible impacts on other Federal project activities.

2.2.1 Land Use

Although the McGuire site is not within the town limits of Huntersville North Carolina (the nearest incorporated town), the site is subject to the extraterritorial zoning jurisdiction of Huntersville. Exercise of extraterritorial jurisdiction is authorized by Section 160A-360 of the General Statutes of North Carolina. The McGuire site is located in a special-purpose zoning district. Power generation plants are a permitted use in special-purpose districts (Town of Huntersville 2001).

2.2.2 Water Use

Lake Norman, North Carolina's largest reservoir, was created by constructing the Cowan's Ford Dam on the Catawba River. Lake Norman is part of the Catawba-Wateree Project, which consists of 11 reservoirs operated for hydroelectric power generation on the Catawba River and licensed by the Federal Energy Regulatory Commission.

In addition to supplying the cooling water for the McGuire plant, Lake Norman also supplies water for Duke Power's coal-fired Marshall Steam Station on the western shore of the lake, approximately 26 km (16 mi) upstream from McGuire. Lake Norman also is a source of

1 municipal drinking water for several cities in the region. Lake Norman supports extensive
2 recreational use by fishermen, boaters, skiers, and swimmers.

3
4 Construction of the Cowan's Ford Dam and impoundment of the Lake Norman reservoir to
5 serve a variety of purposes, including providing cooling water for McGuire, have considerably
6 altered the regional water resources environment. Lake Norman represents the critical
7 landscape feature to lakeside development and regional recreation.

8
9 McGuire employs a once-through cooling system. The average daily withdrawal from Lake
10 Norman for the cooling water and other service water systems is 9580 million L/d
11 (2530 million gpd). The average daily discharge to Lake Norman from McGuire also is
12 approximately 9580 million L/d (2530 million gpd). Approximately 4090 m³/d (1.08 million gpd)
13 from the conventional wastewater treatment system and from the wastewater collection basin
14 are discharged to the Catawba River.

15
16 Potable water at McGuire is supplied by the CMUD water supply system. McGuire has six
17 groundwater wells with a combined maximum pumping rate of 4.3 L/s (68 gpm).

18 19 **2.2.3 Water Quality**

20
21 Lake Norman provides water of sufficiently high quality to serve a variety of needs, including
22 propagation of fish and wildlife and contact recreation. The NCDENR Division of Water Quality
23 found Lake Norman fully supportive of all uses (NCDENR 1999).

24
25 Pursuant to the Federal Water Pollution Control Act of 1977, also known as the Clean Water
26 Act, the water quality of the plant effluents is regulated through the NPDES. The Division of
27 Water Quality within the NCDENR is delegated to issue NPDES permits. The current permit
28 (NC0024392) was issued February 28, 2000, and is due to expire February 28, 2005. Any new
29 regulations promulgated by the U.S. Environmental Protection Agency (EPA) or the State of
30 North Carolina would be reflected in future permits.

31 32 **2.2.4 Air Quality**

33
34 The McGuire site is located in the Piedmont of the Carolinas, a transitional region between the
35 Blue Ridge Mountains to the west and the Coastal Plain to the east. The region has a
36 moderate climate with cool winters and warm summers. Climatological records for Charlotte,
37 North Carolina (NCDC 2001), are generally representative of the McGuire site. Normal daily
38 maximum temperatures for Charlotte range from about 9°C (49°F) in January to a high of

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about 32°C (89°F) in July. Normal minimum temperatures range from about -1°C (30°F) in January to about 21°C (70°F) in July. The average precipitation of about 109 cm (43.1 in.) per year is rather evenly distributed through the year. Normal monthly precipitation ranges from 7 to 11 cm (2.7 to 4.4 in.).

The wind energy resource in the Piedmont of the Carolinas is limited. The annual average wind power in the region is rated 1 on a scale of 1 through 7 (Elliott et al. 1986). Wind turbines are economical in wind power classes 4 through 7 (average wind speeds of 5.6 to 9.4 m/s (12.5 to 21.1 mph) (DOE 2001). The average wind power of exposed coastal areas of North Carolina is rated 3, and the wind power rating for mountain summits and ridges to the west generally varies from 3 to 6.

Thunderstorms can occur in any month and occur on an average of more than 3 days per month from April through August. Hurricanes that strike the Carolina coast may produce heavy rains but seldom cause high winds at the site (NCDC 2001). Statistics for the 30 years from 1954 through 1983 (Ramsdell and Andrews 1986) indicate that the probability of a tornado striking the site is expected to be about 2×10^{-4} per year.

The McGuire site is located within the Metropolitan Charlotte Interstate Air Quality Control Region. This region is designated as in attainment or unclassified for criteria pollutants in 40 CFR 81.334 except for the EPA's reinstated 1-hr ozone standard. Mecklenburg County is a maintenance area for the 1-hr ozone. The State of North Carolina and Mecklenburg County have adopted EPA's proposed 8-hr ozone standard. This standard was exceeded on 32 days in 1999 (Mecklenburg County Department of Environmental Protection [MCDEP] 2000). Monitoring data for Mecklenburg County also indicate that the annual average concentration of fine particles (PM_{2.5}) for 1999 exceeded the PM_{2.5} standard adopted by EPA in 1997. Six areas in North and South Carolina are designated in 40 CFR 81.422 and 40 CFR 81.426 as mandatory Class I Federal areas in which visibility is an important value. All of these Class I areas are more than 80 km (50 mi) from the site.

Diesel generators and other activities and facilities associated with McGuire emit various pollutants. Emissions from these sources are regulated under Air Quality Permit to Construct/Operate 00-019-269 issued by the MCDEP on February 23, 2000.

2.2.5 Aquatic Resources

Aquatic resources in the vicinity of the McGuire site are associated with the southernmost portion of Lake Norman, North Carolina's largest man-made reservoir. In addition to serving McGuire, Lake Norman also provides water to Duke Power's Marshall Steam Station and the Cowan's Ford Dam hydroelectric station. The lake also is a source of drinking water for several

1 cities in the region. Boaters, fishermen, swimmers, and water skiers use the lake for recreation.
2 Centers for tourism and conservation in the vicinity include Lake Norman State Park and three
3 county parks on the shores of the lake. The Cowan's Ford Wildlife Refuge (owned and
4 operated by Mecklenburg County Parks and Recreation Department) and the Cowan's Ford
5 Waterfowl Refuge (managed by North Carolina Wildlife Resources Commission) are located
6 along the shores of Mountain Island Lake, south of the McGuire site and immediately
7 downstream of the Cowan's Ford Dam.

8
9 Lake Norman's major tributaries include the Catawba River, Lyle Creek, and Buffalo Shoals
10 Creek. The lake itself covers 13,150 ha (32,500 ac) and averages 10 m (33 ft) deep, with a
11 maximum 36.6-m (120-ft) depth.

12
13 Pelagic fish species are primarily forage fish, including threadfin shad (*Dorosoma petenense*),
14 gizzard shad (*D. cepedianum*), and alewife (*Alosa aestivalis*). Game fish include black crappie
15 (*Pomoxis nigromaculatus*) and white crappie (*P. annularis*), largemouth bass (*Micropterus*
16 *salmoides*), white perch (*Morone americana*), white bass (*M. chrysops*), striped bass (*M.*
17 *saxatilis*), and some spotted bass (*Micropterus punctulatus*). The primary fish caught in the
18 nearshore littoral zone include sunfish (*Lepomis* spp.), carp (*Cyprinus carpio*), and catfish,
19 including the blue catfish (*Ictalurus furcatus*), snail bullhead (*Ameiurus brunneus*), white catfish
20 (*I. catus*), and flat bullhead (*I. platycephalus*). The blue catfish, white perch, threadfin shad,
21 white bass, spotted bass, and alewife are introduced species, some of which may impact native
22 species populations. In addition, striped bass are not indigenous to Lake Norman and do not
23 reproduce naturally. Instead, they are stocked on an annual basis to provide a resource for
24 sport fishermen.

25
26 In addition to finfish, numerous aquatic invertebrate species are found in the vicinity of McGuire.
27 These include diverse phytoplankton, zooplankton, and benthic macroinvertebrates. In 1999,
28 135 species of phytoplankton were collected, the dominant types being cryptophytes and
29 diatoms (Duke 2001a). Zooplankton communities in Lake Norman also are diverse and tend to
30 fluctuate seasonally and spatially. Since 1987, Duke researchers have observed 108
31 zooplankton taxa (Duke 2001a). Most recently (1999), immature copepods dominated the
32 zooplankton standing crop during most of the year, while rotifers and cladocerans had the
33 highest densities in February and August, respectively. Information from 1977 through 1984
34 indicates that benthos at sublittoral locations was dominated by chironomids, chaoborids,
35 *Corbicula* sp., *Hexagenia* spp., and oligochaetes (Duke Power Company 1985). Since 1989,
36 benthic macroinvertebrate studies have been limited to determining seasonal densities of
37 *Corbicula* sp. in front of the McGuire intake structures. Recent studies indicate that the
38 potential for biofouling from these organisms is moderate to high, but population numbers in
39 front of the intake structures vary widely from year to year (Hall and Wilda 2000, 2001; Duke
40 2001a). Adult clams, capable of reproduction, generally comprise 10 percent or less of the
41 samples (Duke 2001a).

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1 The McGuire site lies entirely in Mecklenburg County. However, Lincoln County, immediately
2 to the west of the site, also could harbor species that would be affected by plant refurbishment
3 or continued operation. A search through the FWS database and the North Carolina National
4 Heritage Program for Federally and State-listed species indicated that two fish—Carolina darter
5 (*Ethostoma collis collis*) and highfin carpsucker (*Carpoides velifer*)— and three mussel
6 species—Carolina heelsplitter (*Lasmigona decorata*), dwarf threetooth (*Triodopsis fulciden*),
7 and Carolina creekshell (*Villosa vaughniana*)—could inhabit the region around McGuire
8 (Table 2-1). In addition, a summer 2000 biological assessment of species associated with
9 McGuire and related power transmission lines (Gaddy 2001) indicated that three other
10 important species, including two mussels—the Carolina elktoe (*Alasmodonta robusta*) and
11 Eastern creekshell (*V. delumbis*)—and one fish—the Santee chub (*Cyprinella zanema*)—could
12 also inhabit the region around McGuire (Table 2-1).

13
14 Gaddy (2001) inventoried the site environs, excluding the industrial areas in the center of the
15 site, using aerial photographs supplemented by field work. Gaddy also walked the four power
16 line rights-of-way in their entirety. Areas that appeared to be reasonable habitat for Federally
17 and State-listed species were inventoried intensively in the summer and the early autumn.
18 Despite an extensive survey program conducted by the State and licensee, no Federal-or State-
19 listed species or critical habitat for such species was found within the McGuire site exclusion
20 area (see Figure 2-4) or along related power transmission rights-of-way (Gaddy 2001).

21
22 Of the species mentioned, only the Carolina heelsplitter is listed as endangered. The other
23 species are considered species of concern or “significantly rare.” The Carolina heelsplitter was
24 known historically in the Catawba River system in Mecklenburg County. However, recent
25 collection records indicate the Carolina heelsplitter has been eliminated from all but one of the
26 streams where it was originally known to exist. In North Carolina, the only remnant populations
27 appear to exist in Union County, far to the southeast of the site (Fridell 2001). All of the
28 streams supporting this species are free-flowing and natural (EPA 2002) and no longer occur in
29 the vicinity of the plant. The last known occurrence in Mecklenburg County was more than 20
30 years in the past (Fridell 2001).

31
32 Menhinick (1991) lists the highfin carpsucker from Lake Norman considerably north of the study
33 area and lists only historic records of the Santee chub in Lake Norman but north of the study
34 area (Gaddy 2001).

Table 2-1. Federal and State of North Carolina Listed Aquatic Species Potentially Occurring in Lincoln and Mecklenburg Counties

Scientific Name	Common Name	Federal Status ^(a)	State Status ^(a)	County
<i>Ethostoma collis collis</i>	Carolina darter	FSC	–	Mecklenburg
<i>Carpoides velifer</i>	highfin carpsucker	–	SC	Mecklenburg
<i>Cyprinella zanema</i>	Santee chub	–	SR	Mecklenburg or Lincoln
<i>Lasmigona decorata</i>	Carolina heelsplitter	E	E	Mecklenburg
<i>Triodopsis fulciden</i>	dwarf threetooth	–	SC	Lincoln
<i>Villosa vaughniana</i>	Carolina creekshell	FSC	SC	Mecklenburg
<i>Villosa delumbis</i>	Eastern creekshell	–	SR	Mecklenburg or Lincoln
<i>Alasmidonta robusta</i>	Carolina elktoe	–	EX	Mecklenburg or Lincoln

(a) E = endangered; EX = extirpated (no longer found in the area); FSC = Federal species of concern; SC = State species of concern but not protected under State regulations; SR = significantly rare but not protected under State regulation; – = no listing.

The three freshwater mussel species – dwarf threetooth, Eastern creekshell, and Carolina creekshell—are not reported from the Lake Norman South quadrangle, according to the North Carolina Natural Heritage Program database <<http://www.ncsparks.net/nhp/search.html>>.

2.2.6 Terrestrial Resources

Forest is the primary land cover near the McGuire site, with pasture, cropland, and residential development each contributing substantially to total land use. Noteworthy natural habitats outside the McGuire site include the 270-ha (668-ac) Cowan's Ford Wildlife Refuge (Figure 2-2) (owned and operated by Mecklenburg County Parks and Recreation Department) and the Cowan's Ford Waterfowl Refuge (Figure 2-2) (managed by the North Carolina Wildlife Resources Commission) to the south along the shores of Mountain Island Lake. These areas, as well as adjacent lands, are occupied by bottomland hardwood forests and other habitats that support nearly 200 species of birds, 54 of which are neotropical migrants. Because of this rich avian diversity, the lands from Cowan's Ford to Mountain Island Lake have been officially designated as IBAs by the National Audubon Society. In addition, wildlife such as wild turkey (*Meleagris gallopavo*), numerous raptor species, whitetail deer (*Odocoileus virginianus*), and red fox (*Vulpes vulpes*) use these IBAs and the properties around the McGuire site to move freely along the Catawba River corridor (Duke 2001a).

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The McGuire exclusion area is a circle with a 760-m (2500-ft) radius (Figure 2-5) that covers 182 ha (450 ac). Two man-made water bodies, the standby nuclear service water pond (13.3 ha [32.9 ac]) and the wastewater collection basin (4.13 ha [10.2 ac]), are located within the exclusion area (Figure 2-5). The exclusion area includes portions of Lake Norman and the McGuire discharge canal. Approximately 58.7 ha (145 ac) of the exclusion area are composed of generation and maintenance facilities, parking lots, roads, storage yards, and mowed grass. The remaining 41.3 ha (102 ac) consist of forest communities (Duke 2001a). In addition, 4.5 km (2.8 mi) of transmission line right-of-way connects the exclusion area to the McGuire switching station via nonforested terrestrial habitat.

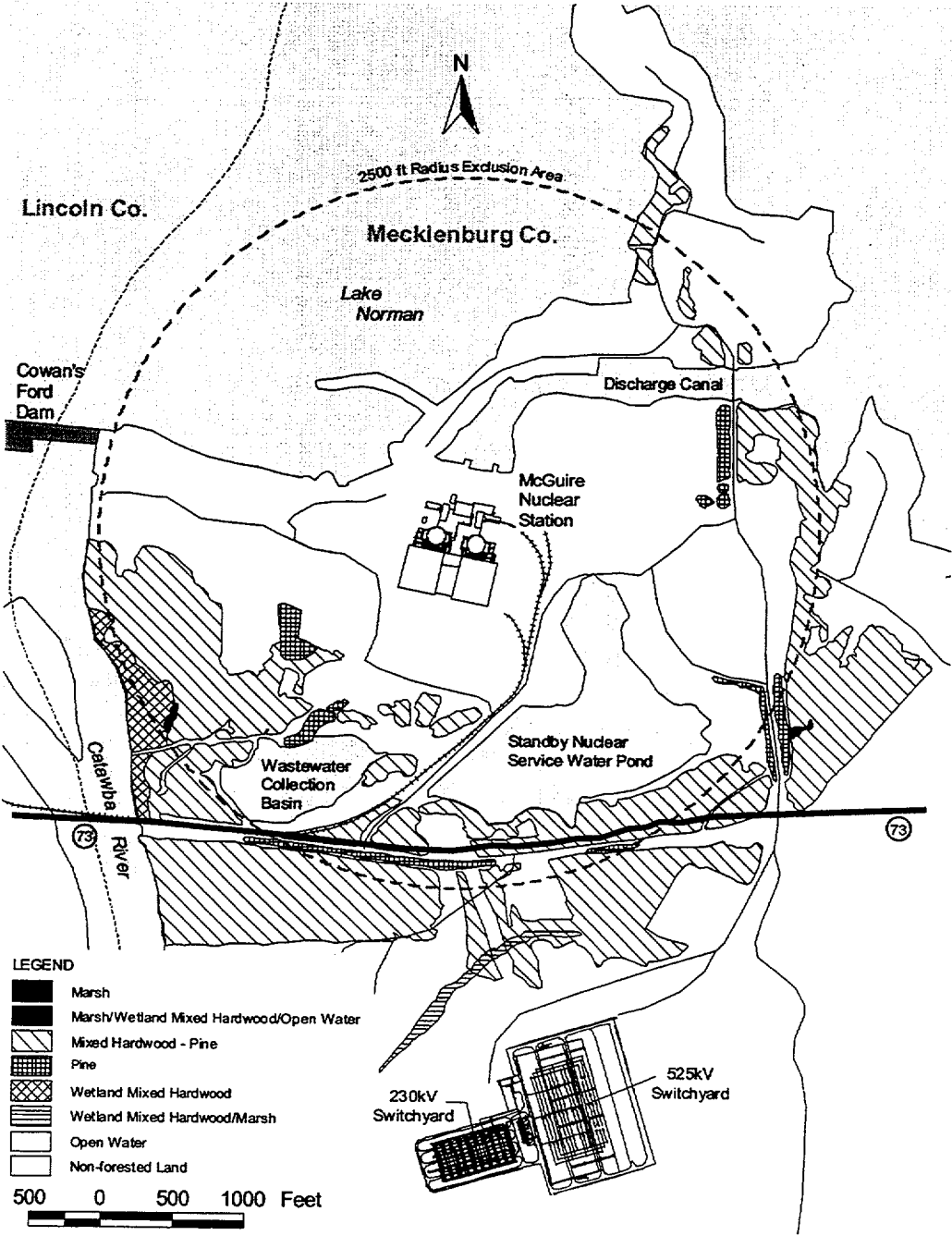
The exclusion area harbors typical Piedmont plant communities (Duke 2001a) and land cover types. As shown in Figure 2-5, seven plant communities or cover types have been identified at the McGuire site: marsh; marsh/wetland mixed hardwood/open water; mixed hardwood-pine; pine; wetland mixed hardwood; wetland mixed hardwood/marsh; and open water (Gaddy 2001). Cecil sandy loam dominates the site, with some Monacan clay loam found along the Catawba River. The more rare and more alkaline Mecklenburg and Iredell soils, which often support prairie plant species, are absent from the site (Duke 2001a; Gaddy 2001).

Marshes are nonforested and found along the margin of the floodplain of the Catawba River. Dominant marsh species include black willow (*Salix nigra*), tag alder (*Alnus serrulata*), a mallow (*Hibiscus* sp.), false nettle (*Boehmeria cylindrica*), fringed sedge (*Carex crinita*), cattail (*Typha latifolia*), rice cut-grass (*Leersia oryzoides*), and the exotic Asiatic dayflower (*Analeima keisak*) (Gaddy 2001).

Marsh/wetland mixed hardwood/open water describes a small wetland altered by beavers (*Castor canadensis*) found along the eastern edge of the exclusion area boundary. Common needlerush (*Juncus effusus*), sedges (*Carex* spp.), and false nettle occur in the backwaters of a small pond on the site. Black willow, tag alder, and sycamore (*Platanus occidentalis*) are found in the wetland mixed hardwood community upstream from the pond (Gaddy 2001).

The mixed hardwood-pine community is the most widespread forest type on the McGuire site. Dominant species include white oak (*Quercus alba*), red oak (*Q. rubra*), tulip poplar (*Liriodendron tulipifera*), post oak (*Q. stellata*), hickories (*Carya* spp.), shortleaf pine (*Pinus echinata*), and Virginia pine (*P. virginiana*). Gaddy (2001) identified a portion of this forest community as a "significant natural area." This area supports a well-developed mixed hardwood forest with scattered mature trees (some greater than 2 ft in diameter). Tulip poplar, white oak, red oak, white ash (*Fraxinus americana*), and hickories dominate the canopy of this area, while dogwood (*Cornus florida*), sourwood (*Oxydendrum arboreum*), strawberry bush (*Calycanthus floridus*), and big-leaved storax (*Styrax grandifolia*) are found in the shrub layer of the understory.

1



2

3

Figure 2-5. McGuire Site Vegetation Types

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1 The pine community is early successional and is dominated by loblolly pine (*P. taeda*) with a
2 low-density groundcover. Most of these stands occur in disturbed areas and along forest edges
3 and appear to have been planted (Gaddy 2001).

4
5 The wetland mixed hardwood community is found in the floodplain of the Catawba River along
6 the western edge of the exclusion area. Dominant overstory species include sweet gum
7 (*Liquidambar styraciflua*), red maple (*Acer rubrum*), American elm (*Ulmus americana*), river
8 birch (*Betula nigra*), and sycamore. Box elder (*A. negundo*) is the understory dominant. The
9 forest floor is occupied by sedges, Japanese honeysuckle (*Lonicera japonica*), and Vietnam
10 grass (*Microstegium vimineum*) (Gaddy 2001).

11
12 The wetland mixed hardwood/marsh community occurs just south of the exclusion area where
13 transmission lines pass over a small tributary of the Catawba River. Sycamore, black willow,
14 tag alder, and sweet gum grow in the forested portions of the wetland, with Vietnam grass and
15 cutgrass (*Leersia* sp.) in the understory. False nettle, common needlerush (*Scirpus*
16 *polyphyllus*), and groundnut (*Apios americana*) grow in marshy openings (Gaddy 2001).

17
18 The forested portion of the exclusion area, as well as the transmission line rights-of-way, do not
19 provide significant terrestrial habitat because of the small acreage involved. However, McGuire
20 site contains man-made wildlife food plots, including strip plots in the rights-of-way, that attract
21 whitetail deer and other wildlife, including songbirds, a variety of mice and voles, raptors, gray
22 fox (*Urocyon cinereoargenteus*), raccoon (*Procyon lotor*), and opossum (*Didelphis virginiana*).

23
24 Food plots include sorghum, sunflowers, rye, clover, and wheat that are mowed selectively to
25 further enhance wildlife habitat value (Duke 2001a).

26
27 Notable wildlife species common to the McGuire site include whitetail deer, wild turkey, Canada
28 geese (*Branta canadensis*), great blue heron (*Ardea herodias*), muskrat (*Ondatra zibethicus*),
29 and osprey (*Pandion haliaetus*). Whitetail deer numbers have increased since McGuire has
30 been operating. This is attributable largely to forest fragmentation, which provides for more
31 open area and an increase in the foraging area for the deer. Fifteen wild turkeys were released
32 on the McGuire site in 1996, and this population is apparently increasing. Wild turkeys are
33 commonly observed frequenting the food plots, rights-of-way, and bottomland hardwood areas.
34 Canada geese numbers around McGuire also are increasing. These, and to a lesser extent
35 other waterfowl and birds, routinely travel between the McGuire site and Cowan's Ford
36 Waterfowl Refuge on Mountain Island Lake. Year-round access to reliable food sources in
37 agricultural settings, yards, golf courses, and other open spaces explains why many of these
38 are nonmigratory. A great blue heron rookery exists on Davidson Creek Island in Lake Norman
39 approximately 4.5 km (3 mi) north of McGuire. This rookery consists of approximately 30 nests
40 and is protected under the North Carolina Wildlife Resources Commission Colonial Waterbird

1 Nesting Area Program. Island access is prohibited from April 1 to August 31. Muskrats,
 2 osprey, and various salamanders, aquatic snakes, and turtles have commonly been observed in
 3 marshy lowland areas and near open water (Duke 2001a).
 4

5 Duke has a progressive wildlife enhancement program for which it received WAIT (Wildlife and
 6 Industry Together) certification from the North Carolina Wildlife Federation in 2001. This
 7 program is implemented both in the relatively unused portions of the plant site and offsite on
 8 nearby properties. It includes establishment and maintenance of food plots in the exclusion
 9 area and the rights-of-way; introduction of wild turkeys in cooperation with the Wild Turkey
 10 Federation; establishment of an osprey hatching site near Cowan's Ford Dam in cooperation
 11 with the Carolina Raptor Center; deeding Davidson Creek Island to the North Carolina Wildlife
 12 Resources Commission for management under the Colonial Waterbird Nesting Area Program;
 13 and establishment of bluebird houses.
 14

15 Eight Federally listed and 10 State-listed threatened or endangered species, candidate species,
 16 or species of special concern are known to occur or may potentially occur in Mecklenburg
 17 County (Table 2-2) (Cole 2001; NCDENR 2001). Bald eagles (*Haliaeetus leucocephalus*) are
 18 known to nest at Lake Wylie (downstream of McGuire) and Lake James (upstream of McGuire)
 19 and are known from the Catawba River area (Cole 2001). The eagles are observed
 20 occasionally along Lake Norman (Cole 2001; Duke 2001a; Gaddy 2001), but sightings are rare
 21 and there are no known nest sites within 100 km (60 mi) of the McGuire site. Except for the
 22 bald eagle, no Federally or State-listed species are known to occur within the McGuire
 23 exclusion area or associated transmission line rights-of-way (Duke 2001a; Gaddy 2001).
 24 However, Schweinitz's sunflower (*Helianthus schweinitzi*) and Georgia aster (*Aster georgianus*)
 25 are known to occur on adjacent property (Cole 2001). No areas designated by the FWS as
 26 critical habitat for threatened/endangered species are known to exist within the McGuire
 27 exclusion area or associated transmission line rights-of-way (Duke 2001a; Gaddy 2001).
 28

29 **2.2.7 Radiological Impacts**

30
 31 Duke has conducted a radiological environmental monitoring program (REMP) around the
 32 McGuire site since 1977 (Duke 2001d). The radiological impacts to workers, the public, and the
 33 environment have been routinely monitored, documented, and compared to the appropriate
 34 standards. The REMP has four key objectives:
 35

- 36 • Provide assurance that McGuire's contribution of radioactivity to the environment is and
 37 remains within applicable limits (Duke 2000a)
- 38
- 39 • Detect and identify changes in environmental levels as a result of station operations
 40 (Duke 2001d)
- 41

Table 2-2. Federal and State of North Carolina-Listed Terrestrial Species Potentially Occurring in Mecklenburg County.

Scientific Name	Common Name	Federal Status ^(a)	State Status ^(a)
BIRDS			
<i>Haliaeetus leucocephalus</i>	bald eagle	T	E
<i>Lanius ludovicianus ludovicianus</i>	loggerhead shrike		SC
MAMMALS			
<i>Condylura cristata</i>	star-nosed mole - coastal plain population		SC
PLANTS			
<i>Aster georgianus</i>	Georgia aster	C	T
<i>Delphinium exaltatum</i>	tall larkspur	FSC	E
<i>Echinacea laevigata</i>	smooth coneflower	E	E
<i>Helianthus schweinitzii</i>	Schweinitz's sunflower	E	E
<i>Isoetes virginica</i>	Virginia quillwort	FSC	C
<i>Lotus helleri</i>	Carolina birdfoot-trefoil	FSC	C
<i>Rhus michauxii</i>	Michaux's sumac	E	E

(a) E = endangered; T = threatened; FSC = Federal species of (special) concern; C = candidate for Federal or State listing; SC = State species of special concern, but not protected under State regulations.

- Provide representative measurements of radiation and radioactive materials in the exposure pathways for the radionuclides that have the highest potential for radiation exposures of members of the public.
- Supplement the radiological effluent monitoring program by verifying that the measurable concentrations of radioactive materials and levels of radiation are not higher than expected on the basis of the effluent measurements and the modeling of the environmental exposure pathways (Duke 2001d).

Radiological releases are summarized in the annual reports—*McGuire Nuclear Station Units 1 and 2—Annual Radiological Environmental Operating Report* (Duke 2001d) and *McGuire Nuclear Station Annual Radioactive Effluent Release Report* (Duke 2000b, 2001c). The limits for all radiological releases are specified in the McGuire ODCM (Duke 2001e), and these limits

are designed to meet Federal standards and requirements. The REMP includes monitoring of the air, direct radiation, surface water, drinking water, shoreline sediment, milk, fish, broadleaf vegetation, and food products.

Review of historical data on releases and the resultant dose calculations revealed that the doses to maximally exposed individuals in the vicinity of the McGuire site were a small fraction of the limits specified in the EPA's environmental radiation standards 40 CFR Part 190 as required by 10 CFR 20.1301(d). For 2000 (the most recent year for which data were available), dose estimates were calculated based on actual liquid and gaseous effluent release data (Duke 2001c) and on measured concentrations of radionuclides from the REMP (Duke 2001d). Dose estimates based on effluent data were performed using the plant effluent release data, onsite meteorological data, and appropriate pathways identified in the ODCM.

A breakdown of maximum dose to an individual located at the McGuire site boundary from effluent-based releases and environmental-based releases for the year 2000 is as follows:

- Total body dose from liquid effluent-based estimates was 0.001 mSv (0.102 mrem) compared to 0.00056 mSv (0.056 mrem) from environmental-based estimates. These estimates were between 1 and 2 percent of the 0.06-mSv (6-mrem) dose limit.^(a) The maximum total organ dose for the liquid effluent-based estimates was 0.0013 mSv (0.13 mrem) to the child liver compared to 0.00064 mSv (0.064 mrem) to the child liver from the environmental-based estimates. These estimates were between 0.32 and 0.65 percent of the 0.20 mSv (20-mrem) dose limit (Duke 2001d).
- The air dose due to noble gases in gaseous effluents was 0.00084 mSv (0.084 mrad) gamma (0.42 percent of the 0.20-mGy [20-mrad] gamma dose limit)^(a) and 0.00031 mGy (0.031 mrad) beta (0.08 percent of the 0.40-mGy [40-mrad] beta dose limit)^(a) (Duke 2001d). Noble gases are not collected as part of the REMP; therefore, an environmental-based estimate was not calculated (Duke 2001d).
- The critical organ dose from gaseous effluents due to iodine-131, iodine-133, tritium, and particulates with half-lives greater than 8 days is 0.0055 mSv (0.55 mrem), which is approximately 2 percent of the 0.30-mSv (30-mrem) dose limit^(a) (Duke 2001d).

Duke does not anticipate any significant changes to the radioactive effluent releases or exposures from McGuire operations during the renewal period, and, therefore, the impacts to the environment are not expected to change.

(a) The dose limit is twice the dose limit in CFR Part 50, Appendix I, because the limit is per reactor unit and McGuire has two operating reactor units.

2.2.8 Socioeconomic Factors

The staff reviewed the McGuire Environmental Report (ER) and information obtained from several county, city, and economic development staff during a site visit from September 24 to 28, 2001. The following information describes the economy, population, and communities near the McGuire site.

2.2.8.1 Housing

Approximately 1370 employees work at McGuire Units 1 and 2. Approximately 23 percent of these employees live in Mecklenburg County, 22 percent live in Lincoln County, 13 percent live in Gaston County, 11 percent live in Iredell County, and the rest live elsewhere in the region (see Table 2-3).

Table 2-3. McGuire Employee Residence Information by County

County	Number of Personnel	Percent	Cumulative Percent
Mecklenburg	318	23	23
Lincoln	305	22	46
Gaston	180	13	59
Iredell	155	11	70
Catawba	121	9	79
Cabarrus	93	7	86
Rowan	63	5	90
South Carolina	63	5	95
Other North Carolina	48	4	98
Other States	21	2	100
Total	1367	100	—
Source: Duke (2001a)			

Duke refuels each nuclear unit at the McGuire site every 18 to 24 months. During these refueling outages, site employment increases by approximately 1015 temporary workers for 30 to 40 days. No major plant refurbishment activities were identified as necessary beyond routine replacement of components as part of normal plant maintenance (Duke 2001a). Duke has no plans to augment its current work force during the term of the license renewal period (Duke 2001a).

Table 2-4 provides the number of housing units, vacancies, vacancy percentages, and 10-year census percentage change for the seven counties in which 90 percent of McGuire employees reside. The vacancy rate for the principal counties of residence is similar, between 5 and 9 percent.

Table 2-4. Housing Units and Housing Units Vacant by County During 1990 and 2000

	1990	2000	Approximate Percentage Change
MECKLENBURG COUNTY			
Housing Units	216,416	292,780	35
Occupied Units	200,219	273,416	37
Percent Vacant	7	7	0
LINCOLN COUNTY			
Housing Units	20,189	25,717	27
Occupied Units	18,764	24,041	28
Percent Vacant	7	7	0
GASTON COUNTY			
Housing Units	69,133	78,842	14
Occupied Units	65,347	73,936	13
Percent Vacant	5	6	20
IREDELL COUNTY			
Housing Units	39,191	51,918	32
Occupied Units	35,573	47,360	33
Percent Vacant	9	9	0
CATAWBA COUNTY			
Housing Units	49,192	59,919	22
Occupied Units	45,700	55,533	22
Percent Vacant	7	7	0
CABARRUS COUNTY			
Housing Units	39,713	52,848	33
Occupied Units	37,515	49,519	32
Percent Vacant	6	6	0
ROWAN COUNTY			
Housing Units	46,264	53,980	17
Occupied Units	45,512	49,940	10
Percent Vacant	8	7	-13

2.2.8.2 Public Services

Public services include utilities (e.g., water supply), education, and transportation.

- **Water Supply**

The CMUD, the largest public water and wastewater utility in the Carolinas, provides drinking water to more than 700,000 people via an estimated 192,000 active water service connections in the City of Charlotte and greater Mecklenburg County—including the towns of Matthews, Mint Hill, Pineville, Huntersville, Davidson, and Cornelius. The drinking water is pumped from the Catawba River—either at Mountain Island Lake or Lake Norman—to one of three treatment plants where the water is cleaned, tested, and pumped into the distribution system. The three plants treat and deliver an average of roughly 386 million L/day (102 million gpd) of water on about half the system's capacity.

Six groundwater wells at McGuire supply certain low-volume needs totaling less than 0.0063 m³/s (100 gpm). The site also has a passive dewatering system for the reactor building and auxiliary buildings. The total water usage at McGuire from CMUD for the year 2000 was 71.4 million liters (18.9 million gallons). Based on this figure, McGuire's average daily consumption of CMUD-supplied potable water was 0.0023 m³/s (0.052 million gpd). CMUD estimates that the average annual system demand will be 7.14 m³/s (163 million gpd) through the year 2030. McGuire's usage is 0.03 percent of the total system usage.

- **Education**

The Charlotte-Mecklenburg schools serve about 106,000 students in 86 elementary, 27 middle, and 16 high schools, as well as 9 special programs, not counting an extensive pre-kindergarten program. There is excess capacity in general for all grade levels except high school, for which enrollment equals capacity. This does not include local school or individual classroom-level allocations, for which there may be space/teacher/resource shortfalls.

- **Transportation**

The McGuire vicinity is served by Interstate 77 (I-77), which enters Mecklenburg County from the north and proceeds southwest through the city of Charlotte and south to Columbia, South Carolina. North Carolina Highway 16 (NC-16) provides north-south travel on the west side of the Catawba River. Sixteen miles west of McGuire, U.S. Highway 321 (US 321) runs north and south through the city of Gastonia. Highway NC-73 runs east and west and passes McGuire at the south end of Lake Norman.

Interstate 85 (I-85) is a major east-west highway that traverses the middle of the county through the city of Charlotte.

The plant is located approximately halfway between NC-16 and I-77. Road access to the McGuire site is via NC-73, a two-lane road for most of its length between NC-16 and I-77. An access railroad enters the site from the south along NC-73.

Duke contacted the North Carolina Department of Transportation (NCDOT) Statewide Planning Branch for information on traffic counts near McGuire. The NCDOT provided Average Annual Daily Traffic (AADT) count data and Level of Service (LOS)^(a) designations for the requested locations (Duke 2001a). The AADTs and LOS designation for roads in the vicinity of McGuire are shown in Figure 2.6. The highest AADT counts are south on NC-16 to NC-73, and then along NC-73 to SR 2145. NC-16 is a major corridor for traffic to and from the Charlotte area. The portion of NC-73 between NC-16 and SR 2145 is a major corridor of travel to Interstate I-77. In summary, the LOS on NC-73 in the vicinity of McGuire is D—a high-density, stable flow in which speed and freedom to maneuver are severely restricted and where small increases in traffic will generally cause operational problems.

Continued growth in population, unrelated to McGuire operations, will likely occur in the areas through the period of the extended license. This growth will necessitate increases in traffic capacity to accommodate the population increase. Traffic planning for the region is conducted by the Mecklenburg-Union Metropolitan Planning Organization (MUMPO). The MUMPO maintains a 20-year planning horizon for transportation improvements in the region (MUMPO 1999). The most recent plan extends to the year 2020 and is reviewed and revised on a 5-year cycle. The current plan does not include improvements to the road system near McGuire.

2.2.8.3 Offsite Land Use

The majority of the land area in the region near McGuire is a mixture of pasture, cropland, forest, and residential development. The shoreline of Lake Norman is developed with both vacation and permanent residences, along with campgrounds, boat launch areas, marinas, golf courses, and small retail establishments. The dominant land uses are residential housing (38 percent) and vacant (44 percent).

Two wildlife refuges are close to the plant site. Cowan's Ford Wildlife Refuge abuts the plant site beginning at the Cowan's Ford Dam and extends south about 11 km (7 mi) along the

(a) LOS is a qualitative measure describing operational conditions within a traffic stream and their perception by motorists (NRC 1996).

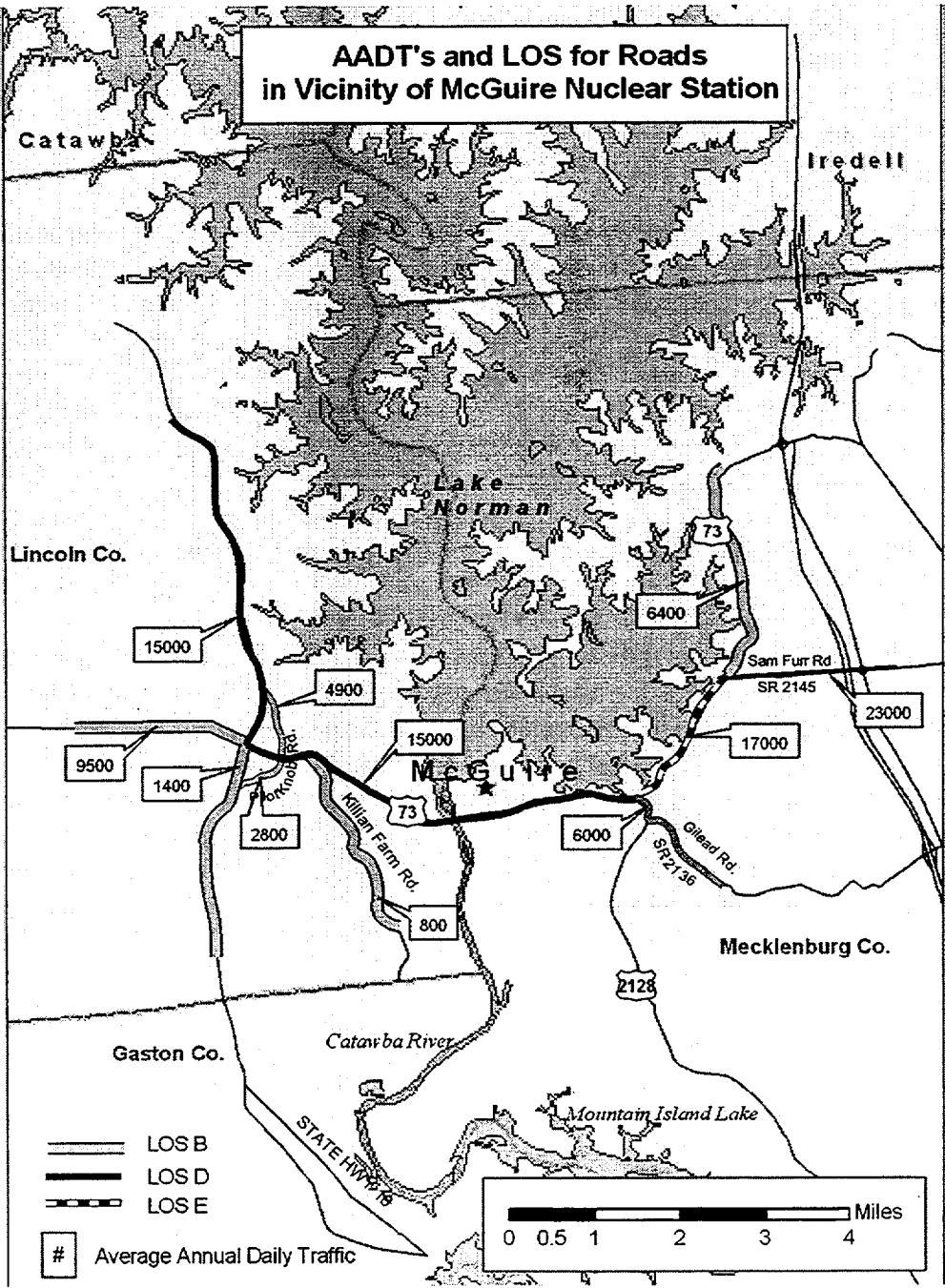


Figure 2-6. Traffic Counts and Level of Service on Roads Surrounding McGuire Nuclear Station

Catawba River. The Cowan's Ford Wildfowl Refuge is about 7 km (4 mi) south of the plant site, within an oxbow bend in the Catawba River just before it flows into Mountain Island Lake. Kings Mountain National Military Park and Kings Mountain State Park are about 48 km (30 mi) southwest of McGuire. South Mountain State Park is approximately 64 km (40 mi) to the west-northwest. Morrow Mountain State Park and a small portion of the Uwharrie National Forest are to the east within an 80-km (50-mi) radius of the McGuire site.

The Catawba Indian Reservation occupies several sites south of the plant near Rock Hill, South Carolina. The nearest of these sites is approximately 48 km (30 mi) from the McGuire site.

2.2.8.4 Visual Aesthetics and Noise

McGuire is visible from a few vantage points on adjoining roads and from Lake Norman. However, its presence does not seem to affect the many recreational boaters or the relatively expensive homes that dot the shoreline. Very little noise from the nuclear station is evident from offsite.

2.2.8.5 Demography

Population was estimated in the region of McGuire in an 80-km (50-mi) zone in 16-km (10-mi) concentric rings. Population estimates for the 80-km (50-mi) area surrounding the site are based on information from the *Updated Final Safety Analysis Report* for McGuire (Duke 2000a).

- **Resident Population Within 80 km (50 mi)**

In 2000, an estimated 2,425,097 people lived within 80 km (50 mi), and 904,943 lived within 32 km (20 mi) of McGuire.

Within 80 km (50 mi) of McGuire are located all or parts of 23 counties in North Carolina and 6 in South Carolina. Within this circle is one major city with a population over 500,000 (2000 Census)—Charlotte, North Carolina. The next largest city is Gastonia, North Carolina, to the southwest, with a population of 66,277 (2000 Census) and Rock Hill, South Carolina, on Highway 21, with a population of 49,765 (2000 Census). Population data for the counties surrounding McGuire (in which 90 percent of McGuire employees live) are shown in Table 2-5.

Table 2-5. Historic and Projected Population in the Principal McGuire Area of Impact - the Seven Counties with 90 Percent of the McGuire Employees

County	1980	1990	2000	2010	2020
Mecklenburg	404,270	511,481	695,454	888,137	1,089,258
Lincoln	42,372	50,319	63,780	77,234	90,778
Gaston	162,568	175,093	190,365	203,623	215,587
Iredell	82,538	92,935	122,660	152,177	182,758
Catawba	105,208	118,412	141,685	163,889	186,058
Cabarrus	85,895	98,935	131,063	164,700	200,092
Rowan	99,186	110,605	130,340	150,599	171,889

Source: 1980 census data available at <http://www.nationalatlas.gov/census1980m.html>. 1990 and 2000 census data available at <http://factfinder.census.gov>. Projections for 2010 and 2020 are available at <http://demog.state.nc.us/>.

• Transient Population

There is very little transient population, either from seasonal travelers or migrant workers, in the vicinity of McGuire (personal communication with Richard W. Jacobsen, Jr., Director, Mecklenburg County Department of Social Services, October 2001; personal communications with Steve Patterson, Charlotte–Mecklenburg Planning Commission, March 2002; personal communication with Donny Hicks, Executive Director, Gaston County Economic Development Commission, March 2002). McGuire is actually in a relatively affluent part of Mecklenburg and surrounding counties, in part because the homes and lots on Lake Norman are considered very desirable.

2.2.8.6 Economy

According to the North Carolina Department of Commerce, Economic Development Information System (available at <http://cmedis.commerce.state.nc.us/region/carolinas.asp>), Mecklenburg County is in the Charlotte Regional Partnership, one of seven economic development regions in North Carolina. Charlotte is the hub of this economic development region. Population growth in Mecklenburg County over the past 20 years is shown in Table 2-5. This region's population and employment grew more rapidly than the state totals in recent years. The largest employment sectors in this region are manufacturing and wholesale/retail trade, while the fastest-growing sectors are construction and services. The business failure rate and business startup rate are slightly below the state average. Per-capita income and average wages are approximately 7 percent above the statewide levels. The unemployment rate is lower than the state average, and the region's poverty rate is the lowest in North Carolina.

Charlotte, the Piedmont Triad, and the Research Triangle region are the state's economic "hot spots," with growth predicted at 19 percent, 17 percent, and 15 percent, respectively, by the

year 2005. Firms such as Hilton Hotels, Marriott Hotels, Hannaford Brothers, Coltec, SeaLand, Omni Hotels, Nations Bank, Hearst Corp., Black & Decker, and Canteen are located in Charlotte. Charlotte's financial sector is also growing and includes Nations Bank and First Union Bank.

Table 2-6 shows the employment by sector and wages in the Mecklenburg area. Table 2-7 shows the employment of the 20 largest manufacturing companies, as reported by the North Carolina Department of Commerce, Economic Development Information System. McGuire's 1370 employees would place it sixth among public and private concerns behind Mecklenburg County itself.

Table 2-6. Employment and Earnings in Key Economic Sectors in Mecklenburg County, North Carolina

	Workforce		Average Weekly Earnings (\$)	
	Number	Percent	County	State
Agriculture	4,864	0.90	472.16	383.00
Construction	32,622	6.30	690.74	571.00
Finance/Insurance/Real Estate (F/I/RE)	58,199	11.30	1,124.78	844.00
Government	48,103	9.40	724.07	621.00
Manufacturing	49,765	9.70	855.04	689.00
Retail Trade	84,054	16.40	409.79	334.00
Wholesale Trade	45,101	8.80	870.05	733.00
Service	145,914	28.40	676.46	550.00
Transportation/Communications/ Public Utilities (T/C/PU)	45,150	8.80	945.34	757.00
Total Workforce^(a)	513,722	100.00		

(a) Mining is excluded because of its very small share of employment in NC and for confidentiality reasons.

Source: North Carolina Department of Commerce, Economic Development Information System available at <http://cmedis.commerce.state.nc.us/countyprofiles/county.profile.asp?county=Mecklenburg>

The unemployment rates for Mecklenburg County and surrounding localities are shown in Table 2-8. Most are below the North Carolina State average of 3.6 percent (U.S. Department of Labor 2001), with the notable exception of Gaston County, reflecting the diverse and healthy economy of the region.

Table 2-7. Twenty Largest Manufacturers in Mecklenburg County

Company	Primary Product Category	Staff
IBM Corp.	Electronic Computers	3000
Solelectron Technology Inc.	Printed Circuit Boards	2500
Continental General Tire Inc.	Tires and Inner Tubes	1700
Lance Inc.	Potato Chips and Similar Products	1600
Microsoft Corp.	Prepackaged Software	1300
Knight Publishing Co.	Newspapers: Publishing and Printing	1000
Interstate Brands Corp.	Bread, Bakery Products Except Cookies and Crackers	900
Frito-Lay Inc.	Potato Chips and Similar Products	720
Clariant Corp.	Cyclic-Crudes, Intermediates, Dyes and Org. Pigments	650
Siemens Westinghouse Power	Steam, Gas, and Hydraulic Turbines and Engines	610
Charlotte Pipe and Foundry Co.	Gray Iron Foundries	520
Blythe Construction Inc.	Commercial Physical and Biological Research	500
Connor, Wilton Packaging Limited Liability Company	Corrugated and Solid Fiber Boxes	500
Hoechst Celanese Corp.	Commercial Physical and Biological Research	500
Continental General Tire Inc.	Tires and Inner Tubes	400
Compass Group North America	Food Preparations	400
Carolina Tractor/Equipment Co.	Machinery and Equipment, Industrial and Commercial	400
AmeriSteel Corp.	Blast Furnaces, Coke Ovens, Steel and Rolling Mills	400
Okuma Machine Tools Inc.	Machine Tool Accessories	400
Conbraco Industries Inc.	Valves and Pipe Fittings	350

Source: North Carolina Department of Commerce, Economic Development Information System available at <http://cmedis.commerce.state.nc.us/countyprofiles/countyprofile.asp?county=Mecklenburg>

McGuire paid about \$8.5 million in property taxes to both Mecklenburg County and the town of Huntersville in fiscal year 1998-99. This represents about 2 percent of the property tax revenue and about 1 percent of the total operating budget of Mecklenburg County. McGuire also pays \$333,333 per year to Huntersville, representing 7 percent of its property tax and 4 percent of its operating budget, as shown in Table 2-9.

Table 2-8. Unemployment in Counties Surrounding McGuire

County	2000 Annual Unemployment Rates (%)
Cabarrus	2.6
Catawba	2.2
Gaston	6.1
Iredell	3.3
Lincoln	4.1
Mecklenburg	2.5
Rowan	4.8
State of North Carolina	3.6
Source: U.S. Department of Labor, Bureau of Labor Statistics, 2000 data (DOL 2001)	

Table 2-9. Property Tax Revenues Generated in Mecklenburg County: 1998-2001^(a)

Tax or Fiscal Year	Total Mecklenburg County Property Tax Revenues (\$) ^(b)	Property Tax Paid to Mecklenburg County by McGuire (\$) ^(c)	McGuire Property Taxes as a Percentage of Total County Property Tax Revenue	Total County Operating Budget (\$) ^(b)	McGuire Property Taxes as a Percentage of Total County Operating Budget
1998	385,673,079	8,100,866	2	760,190,762	1
1999	399,009,088	7,624,712	2	850,502,587	1
2000	445,135,437	7,421,517	2	940,575,290	1
2001	473,588,913	9,311,874	2	1,029,528,662	1

(a) In addition, McGuire pays \$333,333 a year to the town of Huntersville, a part of an agreement for payments in lieu of annexation of the McGuire site by the town of Huntersville. The payments will be made on an annual basis until the year 2027, when the agreement expires. The total revenues received in 1999 by the town of Huntersville were \$9,462,699, of which \$4,832,573 were revenues from property taxes (Duke 2001a, Section 4.18) The payment by McGuire represents about 7 percent of Huntersville's property tax revenue and 4 percent of its total operating budget.

(b) Source: Personal communication from Mecklenburg-Charlotte Tax Assessor, February 2002

(c) Source: Personal communication from North Carolina Department of Revenue, Property Tax Division, March 2002

2.2.9 Historic and Archaeological Resources

This section discusses the cultural background and the known historic and archaeological resources at McGuire and in the surrounding area. This section draws on information contained in the McGuire ER (Duke 2001a) and from archives and records stored at the North

1 Carolina Department of Cultural Resources, Office of Archives and History, as well as published
2 literature that treats the history of the North Carolina Piedmont (Piedmont).

3 4 **2.2.9.1 Cultural Background**

5
6 McGuire is in the southwest section of the Piedmont geologic province. The Piedmont is a
7 large, highly dissected plateau covering some 58,000 km² (20,000 mi²) between the coastal
8 plain and the foothills of the Blue Ridge Mountains (Ward 1983). The Piedmont has an
9 archaeological sequence that extends back at least 12,000 years before the present.

10
11 The Piedmont's cultural history can be divided into five major periods: Paleoindian (10,000
12 B.C., and perhaps as early as 13,000 B.C., to around 8000 B.C.), Archaic (8000 to 500 B.C.),
13 Woodland (500 B.C. to around A.D. 1000), Mississippian (A.D. 1000 to around 1500), and
14 Historic and Modern (A.D. 1500 to the present).

15
16 During the Paleoindian period, the native peoples seemingly were organized into small mobile
17 bands with a hunting- and a fishing-based economy. Animals hunted included megafauna,
18 such as the now extinct mammoth. The environment of the Paleoindian period was significantly
19 different from the present. This was at the end of the last ice age, in which the climate was
20 cooler than at present and glaciers covered much of the northern portion of North America.

21
22 The transition between the Paleoindian and Archaic periods was accompanied by substantial
23 environmental change. As glaciers began to melt, sea level began to rise. These changing
24 environmental conditions led to a greater dependence on river systems and the beginnings of
25 the use of domesticated plants. Middle Archaic sites in the Piedmont are numerous and likely
26 reflect small groups of socially noncomplex peoples widely ranging across the landscape
27 (Anderson 1996). Middle and Late Archaic archaeological sites typically exhibit greater
28 evidence of sedentary economies, such as the presence of storage pits, extensive refuse
29 middens, and large quantities of fire-cracked rock. Archaic period habitation sites appear to
30 have been divided into base camps used during the the spring, summer, and winter months,
31 and smaller upland sites used during the fall for deer hunting and nut gathering.

32
33 In the Woodland period, Native American cultures reached their modern configurations as
34 noted at the time of initial European contact in the 16th and 17th centuries. The middle of the
35 Woodland period witnessed the establishment of large sedentary base camps in river valleys,
36 with associated smaller resource-gathering sites being established in surrounding areas.

Toward the end of the Woodland period and during the subsequent Mississippian period, Native American villages throughout the Midwest and much of the Southeast apparently were organized into chiefdom-level societies (Bense 1994; Perdue 1985). The use of long-houses, palisades, earth lodges, mounds and other earthen works, and designated burial grounds are hallmarks of the Mississippian period.

The staff assumes that the ancestors of the modern Catawba Indians lived in the region surrounding McGuire and the Catawba River at the time of historic contact with the Europeans (Perdue 1985; Merrell 1989; Lee 1997; De Vorse 1998). The Catawba are an eastern Siouan-speaking tribe who likely lived in the Carolinas for several hundred years before European contact.

The Historic period in North Carolina began in the early 16th century with the first incursions of European explorers along the Carolina Coast (Bense 1994; Cumming 1998; De Vorse 1998). Beginning around 1660, a steady stream of Euroamericans began moving from Virginia into the coastal sounds and rivers of North Carolina (Perdue 1985; Lee 1997). In 1670, the Carolina colony was established by the British at Charles Town (modern Charleston). The stream of settlers finally led to a series of conflicts between the tribes and the settlers, with the most serious being the Tuscarora, Yamasse, and Cheraw Wars of 1711-1718. In these wars, the Catawba first assisted the Euroamericans against Tuscarora and then turned on the Euroamericans, particularly in the Yamasse War. Ultimately, the Catawba joined the Cherokee in making peace.

In 1701, the surveyor John Lawson reported that several thousand Catawba Indians were observed living in many different villages (Perdue 1985; Lee 1997). By 1738, smallpox and other diseases had reduced the tribe to around 1000 people living in six villages in proximity along the Catawba River in the area around the present border between South and North Carolina. A second smallpox epidemic in 1759-1760 further reduced the Catawba population.

By 1750, so many Euroamericans had moved into the Piedmont that Anson County was created, a county which then covered roughly the western half of North Carolina. Mecklenburg County itself was carved out from Anson County and established in 1763. The current county boundaries were set up in 1842. Treaties in 1760 and 1763 set up an approximately 39-km² (15-mi²) reservation for the Catawba tribe at the eastern edge of South Carolina; however, these lands were soon overrun by Euroamerican colonists. In 1768, the town of Charlotte was incorporated at the juncture of two major transportation and trade routes (Rogers and Rogers 1996). John Collet's detailed 1770 map of North Carolina (Cumming 1998, Plates), depicts Charlotte (Charlottesburgh) and the small nearby Catawba Tribal Reservation but depicts no settlements, mills, or transportation corridors in the general vicinity of McGuire.

Plant and the Environment

1 In early 1779, the British concentrated on consolidating their power in the southern states
2 during the American Revolution. Charles, First Lord Cornwallis, entered Charlotte on
3 September 28, 1780. However, his reception was so contested that Lord Cornwallis retreated
4 from Charlotte to Charleston on October 14, 1780.

5
6 In December 1780, Nathanael Greene, the commanding general for the Continental Army in the
7 South, arrived in Charlotte. Greene decided that the Charlotte area did not contain enough
8 provisions to satisfactorily supply his army, so he removed the majority of the Army to the Pee
9 Dee River to the east of Charlotte. Some 1000 men under the command of General Daniel
10 Morgan were sent to northwest South Carolina. The British general, Lord Cornwallis began to
11 pursue Morgan, who was fleeing east to attempt to rejoin with Greene. Greene, riding west
12 from his camp, met Morgan at the Catawba River, and was joined by General William Lee
13 Davidson, the local militia commander for the area.

14
15 Because there were no bridges crossing the Catawba River, Davidson and a small force were
16 tasked to slow the advance of the British Army so that Morgan's forces would have time to join
17 up with those of Greene. Just before daybreak the next morning, the British Army led by
18 Cornwallis surprised Davidson's sleeping militia at Cowan's Ford. This was to prove the last bit
19 of local action for the Charlotte area during the American Revolution.

20
21 During the period between the American Revolution and the Civil War, the Piedmont became
22 divided into regions devoted to tobacco growing (north and east of Charlotte) and cotton
23 growing (around and to the south of Charlotte). The period of 1789 through 1860 saw the
24 development of plantations (primarily using African slaves for labor), independent farms, and
25 small towns through the Piedmont, in which agriculture dominated local economies. This was
26 facilitated by the invention of the cotton gin in 1793, which allowed short-fiber cotton to be
27 grown virtually anywhere in the region.

28
29 The Catawba Indians were active resisters to the forced relocation plans of the Federal
30 government during the 1820s to 1850s, such as President Andrew Jackson's Indian Removal
31 Act of 1830 (Bense 1994). The Catawba attempted to hang onto their old reservation lands
32 ceded in the 18th century, but in 1840 were finally forced to sell most of them to South Carolina.
33 The Catawba then variously lived with the North Carolina Cherokee and the Oklahoma Choctaw
34 and then surreptitiously returned to South Carolina.

35
36 The Charlotte area and the Mecklenberg County portion of the Catawba River did not play a
37 major role in the battles and strategy of the Civil War (Barrett 1987). Some Catawba soldiers
38 fought for the Confederacy during the Civil War.

1 Due to the physical effects of the Civil War and to the abolishment of slavery, the economic
2 basis of the Southeast was fundamentally changed between 1865 and 1917 (Bense 1994).
3 While plantations were typically returned to their former owners, plant operations became
4 dependent on voluntary contracts or tenant farming with their labor force. Over time,
5 plantations became smaller, averaging less than 40 ha (100 ac) by 1920. The expansion of the
6 railroads, the rebuilding of basic infrastructure, and the Industrial Revolution all led to major
7 changes.

8
9 The period between World War I and World War II saw the continued growth of small towns
10 and the continuation of the use of small plantations and independent farms. In 1941, the
11 Catawba Tribe first received Federal recognition but petitioned to terminate their status in 1959,
12 with lands being distributed among tribal members (Merrell 1989). After a period of
13 reassessing this decision to divest, the tribal council was reorganized and in 1973 was given
14 state recognition by South Carolina. After a lengthy court process, Federal recognition was
15 reinstated in 1994.

16
17 The Modern period since the end of World War II has witnessed the creation of Lake Norman,
18 North Carolina's largest man-made lake, which reached full capacity in 1964. As a conse-
19 quence, numerous residential developments have blossomed around its margins, a trend that is
20 ongoing. Construction began in the mid-1970s on McGuire Units 1 and 2, and in 1981 and
21 1984, respectively, the units were put into operation.

22 23 **2.2.9.2 Historic and Archaeological Resources at the McGuire Site**

24
25 Historic and archaeological site file searches were conducted at the North Carolina Department
26 of Cultural Resources, Office of Archives and History, to determine what specific historic
27 cultural resources may be present at the McGuire site. In addition, record searches were
28 conducted for nearby locations to gain a perspective on the types of historic resources that may
29 be present in the previously undeveloped and unsurveyed portions of the grounds of the
30 McGuire Nuclear Station.

31
32 These record searches revealed that there are no known historic and archaeological resources
33 at McGuire. During the construction of McGuire, a forgotten historic marker commemorating
34 the death of General Davidson at Cowan's Ford was discovered (Duke 2001a). Cowan's Ford
35 and the location of Davidson's death are now inundated. General Davidson's body was interred
36 at the Hopewell Church cemetery about 8 km (5 mi) away. In 1971, Duke incorporated this
37 marker, as well as a new marker provided by the North Carolina Department of Archives and
38 History, into a public area adjacent to McGuire. The markers were dedicated in 1971 and are
39 still maintained by Duke.
40

1 An archaeological survey was not conducted at McGuire before construction activities.
2 However, based on the records of nearby sites and properties, it is unlikely that significant
3 Native American resources were present. A number of Native American archaeological sites
4 were identified and recorded in the early 1960s just north of McGuire before the creation of
5 Lake Norman. These sites were poorly defined and described but appear to represent Archaic,
6 Woodland, and Mississippian period occupations. Most consisted of a few scattered stone and
7 ceramic artifacts in areas heavily disturbed by historic agriculture, specifically from the
8 cultivation of cotton. Erosion caused by cotton farming was a major impact in virtually every
9 site, with many of the sites being exposed to bedrock.

10
11 No structures or buildings at McGuire are 50 years of age or older. A number of structures and
12 buildings within a 5.0-km (3.1-mi) radius of McGuire have been evaluated for historic
13 significance; however, only three of these have been determined eligible for listing in the
14 National Register of Historic Places (Duke 2001a). These include the Ingleside house, about
15 3.7 km (2.3 mi) from McGuire, which was built in the 1850s; the Rural Hill Plantation, about 4.6
16 km (2.8 mi) from McGuire, which has features dating to the late 18th century; and the Holly
17 Bend house, about 4.9 km (3.0 mi) from McGuire, which was built at the end of the 18th
18 century. The Gilead Associated Reformed Presbyterian church and cemetery and the Caldwell-
19 Rosenwald School are currently pending evaluation.

20
21 The Catawba Indian Reservation (in three separate parcels) is situated in South Carolina about
22 48 km (30 mi) south of McGuire.

23 **2.2.10 Related Federal Project Activities and Consultations**

24
25
26 The staff reviewed the possibility that activities of other Federal agencies might impact the
27 renewal of the OLs for McGuire. Any such activities could result in cumulative environmental
28 impacts and the possible need for the Federal agency to become a cooperating agency in
29 preparing the SEIS [10 CFR 51.10(b)(2)].

30
31 The Federal Power Commission, now the FERC, issued a license (FERC Project No. 2232) to
32 Duke Power Company on September 17, 1958, for the Catawba-Wateree hydroelectric project
33 (FERC 2001a). One component of the project is the Cowan's Ford Dam hydroelectric station.
34 The Cowan's Ford Dam impounds Lake Norman. The license for the Catawba-Wateree project
35 will expire August 31, 2008 (FERC 2001a). Under current FERC rules, Duke Power will need to
36 file a notice of intent with FERC by August 2003 declaring whether or not it intends to seek a
37 new license for the Catawba-Wateree hydroelectric project (18 CFR 16.6). Assuming that Duke
38 Power intends to seek a new license, it will need to file an application for the relicensing of the
39 project at least two years before the license expires. FERC will prepare an environmental

assessment or an EIS under NEPA in conjunction with reviewing the application. FERC's procedures for processing a license application are set out in a handbook (FERC 2001b).

The Federal lands closest to McGuire are within the Kings Mountain National Military Park. The park is located near Blacksburg, South Carolina, and is operated by the National Park Service. The park is approximately 48 km (30 mi) southwest of McGuire.

The Native American land closest to the McGuire site is the Catawba Indian Reservation. The tribe occupies a 260-ha (640-ac) reservation in York County, South Carolina, near the city of Rock Hill. The reservation is approximately 48 km (30 mi) south of McGuire.

Duke's Catawba Nuclear Sation is located approximately 48 km (30 mi) south of McGuire. Duke has requested that the NRC renew the OLs for the Catawba plant also.

After reviewing the Federal activities in the vicinity of McGuire, the staff determined that no Federal project activities could result in cumulative impacts or would make it desirable for another Federal agency to become a cooperating agency for preparing the SEIS.

The NRC is required under Section 102 of NEPA to consult with and obtain the comments of any Federal agency that has jurisdiction by law or special expertise with respect to any environmental impact involved. During the preparation of this draft SEIS, the NRC staff consulted with the FWS. The consultation correspondence is included in Appendix E.

2.3 References

10 CFR 20. Code of Federal Regulations, Title 10, *Energy*, Part 20, "Standards for Protection Against Radiation."

10 CFR 50. Code of Federal Regulations, Title 10, *Energy*, Part 50, "Domestic Licensing of Production and Utilization Facilities."

10 CFR 54. Code of Federal Regulations, Title 10, *Energy*, Part 54, "Requirements for Renewal of Operating Licenses for Nuclear Power Plants."

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1 10 CFR 42. Code of Federal Regulations, Title 10, *Energy*, Part 72, "Licensing Requirements
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3.0 Environmental Impacts of Refurbishment

Environmental issues associated with refurbishment activities are discussed in the *Generic Environmental Impact Statement for License Renewal of Nuclear Plants* (GEIS), NUREG-1437, Volumes 1 and 2 (NRC 1996, 1999).^(a) The GEIS includes a determination of whether the analysis of the environmental issues could be applied to all plants and whether additional mitigation measures would be warranted. Issues are then assigned a Category 1 or a Category 2 designation. As set forth in the GEIS, Category 1 issues are those that meet all of the following criteria:

- (1) The environmental impacts associated with the issue have been determined to apply either to all plants or, for some issues, to plants having a specific type of cooling system or other specified plant or site characteristic.
- (2) A single significance level (i.e., SMALL, MODERATE, or LARGE) has been assigned to the impacts (except for collective offsite radiological impacts from the fuel cycle and from high-level waste and spent fuel disposal).
- (3) Mitigation of adverse impacts associated with the issue has been considered in the analysis, and it has been determined that additional plant-specific mitigation measures are likely not to be sufficiently beneficial to warrant implementation.

For issues that meet the three Category 1 criteria, no additional plant-specific analysis is required unless new and significant information is identified.

Category 2 issues are those that did not meet one or more of the criteria of Category 1 and, therefore, additional plant-specific review of these issues is required.

License renewal actions may require refurbishment activities for the extended plant life. These actions may have an impact on the environment that requires evaluation, depending on the type of action and the plant-specific design. Environmental issues associated with refurbishment that were determined to be Category 1 issues are listed in Table 3-1.

Environmental issues related to refurbishment considered in the GEIS for which these conclusions could not be reached for all plants, or for specific classes of plants, are Category 2 issues. These are listed in Table 3-2.

(a) The GEIS was originally issued in 1996. Addendum 1 to the GEIS was issued in 1999. Hereafter, all references to the "GEIS" include the GEIS and its Addendum 1.

Environmental Impacts of Refurbishment

Table 3-1. Category 1 Issues for Refurbishment Evaluation

ISSUE—10 CFR Part 51, Subpart A, Appendix B, Table B-1	GEIS Sections
SURFACE-WATER QUALITY, HYDROLOGY, AND USE (FOR ALL PLANTS)	
Impacts of refurbishment on surface-water quality	3.4.1
Impacts of refurbishment on surface-water use	3.4.1
AQUATIC ECOLOGY (FOR ALL PLANTS)	
Refurbishment	3.5
GROUNDWATER USE AND QUALITY	
Impacts of refurbishment on groundwater use and quality	3.4.2
LAND USE	
Onsite land use	3.2
HUMAN HEALTH	
Radiation exposures to the public during refurbishment	3.8.1
Occupational radiation exposures during refurbishment	3.8.2
SOCIOECONOMICS	
Public services: public safety, social services, and tourism and recreation	3.7.4; 3.7.4.3; 3.7.4.4; 3.7.4.6
Aesthetic impacts (refurbishment)	3.7.8

Category 1 and Category 2 issues related to refurbishment that are not applicable to McGuire Nuclear Station, Units 1 and 2 (McGuire) because they are related to plant design features or site characteristics not found at McGuire are listed in Appendix F.

10 CFR 54.21 describes a required review to demonstrate that the effects of aging will be managed such that the structure and component intended functions will be maintained consistent with the current licensing basis during the period of extended operations. Duke Energy Corporation (Duke) provided this review in the Technical Information portion of its application for license renewal (Duke 2001). Duke stated that "Based on this review, no major plant refurbishment activities were identified as necessary to maintain the structure and component intended functions consistent with the current licensing basis during the period of extended operations." Therefore, the staff does not further consider refurbishment in this supplement environmental impact statement.

Table 3-2. Category 2 Issues for Refurbishment Evaluation

ISSUE—10 CFR Part 51, Subpart A, Appendix B, Table B-1	GEIS Section	10 CFR 51.53 (c)(3)(ii) Subparagraph
TERRESTRIAL RESOURCES		
Refurbishment impacts	3.6	E
THREATENED OR ENDANGERED SPECIES (FOR ALL PLANTS)		
Threatened or endangered species	3.9	E
AIR QUALITY		
Air quality during refurbishment (nonattainment and maintenance areas)	3.3	F
SOCIOECONOMICS		
Housing impacts	3.7.2	I
Public services: public utilities	3.7.4.5	I
Public services: education (refurbishment)	3.7.4.1	I
Offsite land use (refurbishment)	3.7.5	I
Public services: transportation	3.7.4.2	J
Historic and archaeological resources	3.7.7	K
ENVIRONMENTAL JUSTICE		
Environmental justice	Not addressed ^(a)	Not addressed ^(a)
(a) Guidance related to environmental justice was not in place at the time the GEIS and the associated revision to 10 CFR Part 51 were prepared. If an applicant plans to undertake refurbishment activities for license renewal, environmental justice must be addressed in the applicant's environmental report and the staff's environmental impact statement.		

3.1 References

10 CFR 51. Code of Federal Regulations, Title 10, *Energy*, Part 51, "Environmental Protection Regulations for Domestic Licensing and Related Regulatory Functions."

10 CFR 54. Code of Federal Regulations, Title 10, *Energy*, Part 54, "Requirements for Renewal of Operating Licenses for Nuclear Power Plants."

Environmental Impacts of Refurbishment

1 Duke Energy Corporation (Duke). 2001. *Application to Renew the Operating Licenses of*
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4.0 Environmental Impacts of Operation

Environmental issues associated with plant operations during the renewal term are discussed in the *Generic Environmental Impact Statement for License Renewal of Nuclear Plants (GEIS)*, NUREG-1437, Volumes 1 and 2 (NRC 1996, 1999b).^(a) The GEIS includes a determination of whether the analysis of the environmental issues could be applied to all plants and whether additional mitigation measures would be warranted. Issues were then assigned a Category 1 or a Category 2 designation. As set forth in the GEIS, Category 1 issues are those that meet all of the following criteria:

- (1) The environmental impacts associated with the issue have been determined to apply either to all plants or, for some issues, to plants having a specific type of cooling system or other specified plant or site characteristics.
- (2) A single significance level (i.e., SMALL, MODERATE, or LARGE) has been assigned to the impacts (except for collective offsite radiological impacts from the fuel cycle and from high-level waste and spent fuel disposal).
- (3) Mitigation of adverse impacts associated with the issue has been considered in the analysis, and it has been determined that additional plant-specific mitigation measures are likely not to be sufficiently beneficial to warrant implementation.

For issues that meet the three Category 1 criteria, no additional plant-specific analysis is required unless new and significant information is identified.

Category 2 issues are those that did not meet one or more of the criteria of Category 1 and, therefore, additional plant-specific review of these issues is required.

This chapter addresses the issues related to operation during the renewal term that are listed in Table B-1 of 10 CFR Part 51, Subpart A, Appendix B, and are applicable to McGuire Nuclear Station, Units 1 and 2 (McGuire). Section 4.1 addresses the issues applicable to the McGuire cooling water systems. Section 4.2 addresses issues related to transmission lines and land use. Section 4.3 addresses the radiological impacts of normal operation. Section 4.4 addresses issues related to the socioeconomic impacts of normal operation during the renewal term. Section 4.5 addresses issues related to groundwater use and quality. Section 4.6 discusses the impacts of renewal-term operations on threatened and endangered species. Section 4.7

(a) The GEIS was originally issued in 1996. Addendum 1 to the GEIS was issued in 1999. Hereafter, all references to the "GEIS" include the GEIS and its Addendum 1.

addresses new information that was raised during the scoping period. The results of the evaluation of environmental issues related to operation during the renewal term are summarized in Section 4.8. Finally, Section 4.9 lists the references for Chapter 4. Appendix F lists Category 1 and Category 2 issues that are not applicable to McGuire Nuclear Station, Units 1 and 2 because they are related to plant design features or site characteristics not found at McGuire.

4.1 Cooling System

Category 1 issues in Table B-1 of 10 CFR Part 51, Subpart A, Appendix B, that are applicable to cooling system operation for McGuire during the renewal term are listed in Table 4-1. Duke Energy Corporation (Duke) stated in its environmental report (ER) that "no new information existed for the issues that would invalidate the GEIS conclusions" (Duke 2001a). The staff has not identified any significant new information during its independent review of the McGuire ER (Duke 2001a), the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts related to these issues beyond those discussed in the GEIS. For all of the issues, the staff concluded in the GEIS that the impacts are SMALL, and additional plant-specific mitigation measures are not likely to be sufficiently beneficial to be warranted.

A brief description of the staff's review and the GEIS conclusions, as codified in Table B-1, for each of these issues follows:

- Altered current patterns at intake and discharge structures. Based on information in the GEIS, the Commission found that

Altered current patterns have not been found to be a problem at operating nuclear power plants and are not expected to be a problem during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of altered current patterns during the renewal term beyond those discussed in the GEIS.

- Altered thermal stratification of lakes. Based on information in the GEIS, the Commission found that

Generally, lake stratification has not been found to be a problem at operating nuclear power plants and is not expected to be a problem during the license renewal term.

Table 4-1. Category 1 Issues Applicable to Operation of the McGuire Cooling System During the Renewal Term

ISSUE—10 CFR Part 51, Subpart A, Appendix B, Table B-1	GEIS Sections
SURFACE WATER QUALITY, HYDROLOGY, AND USE (FOR ALL PLANTS)	
Altered current patterns at intake and discharge structures	4.2.1.2.1; 4.3.2.2; 4.4.2
Altered thermal stratification of lakes	4.2.1.2.3; 4.4.2.2
Temperature effects on sediment transport capacity	4.2.1.2.3; 4.4.2.2
Scouring caused by discharged cooling water	4.2.1.2.3; 4.4.2.2
Eutrophication	4.2.1.2.3; 4.4.2.2
Discharge of chlorine or other biocides	4.2.1.2.4; 4.4.2.2
Discharge of sanitary wastes and minor chemical spills	4.2.1.2.4; 4.4.2.2
Discharge of other metals in wastewater	4.2.1.2.4; 4.3.2.2; 4.4.2.2
Water use conflicts (plants with once-through cooling systems)	4.2.1.3
AQUATIC ECOLOGY (FOR ALL PLANTS)	
Accumulation of contaminants in sediments or biota	4.2.1.2.4; 4.3.3; 4.4.3; 4.4.2.2
Entrainment of phytoplankton and zooplankton	4.2.2.1.1; 4.3.3; 4.4.3
Cold shock	4.2.2.1.5; 4.3.3; 4.4.3
Thermal plume barrier to migrating fish	4.2.2.1.6; 4.4.3
Distribution of aquatic organisms	4.2.2.1.6; 4.4.3
Premature emergence of aquatic insects	4.2.2.1.7; 4.4.3
Gas supersaturation (gas bubble disease)	4.2.2.1.8; 4.4.3
Low dissolved oxygen in the discharge	4.2.2.1.9; 4.3.3; 4.4.3
Losses from predation, parasitism, and disease among organisms exposed to sublethal stresses	4.2.2.1.10; 4.4.3
Stimulation of nuisance organisms	4.2.2.1.11; 4.4.3
HUMAN HEALTH	
Microbial organisms (occupational health)	4.3.6
Noise	4.3.7

Environmental Impacts of Operation

1 The staff has not identified any significant new information during its independent review of
2 the McGuire ER, the staff's site visit, the scoping process, its review of monitoring programs,
3 or its evaluation of other available information. Therefore, the staff concludes that there are
4 no impacts of altered thermal stratification of Lake Norman during the renewal term beyond
5 those discussed in the GEIS.

- 6
7 • Temperature effects on sediment transport capacity. Based on information in the GEIS,
8 the Commission found that

9
10 These effects have not been found to be a problem at operating nuclear power
11 plants and are not expected to be a problem during the license renewal term.

12
13 The staff has not identified any significant new information during its independent review of
14 the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available
15 information. Therefore, the staff concludes that there are no impacts of temperature on
16 sediment transport capacity during the renewal term beyond those discussed in the GEIS.

- 17
18 • Scouring caused by discharged cooling water. Based on information in the GEIS, the
19 Commission found that

20
21 Scouring has not been found to be a problem at most operating nuclear power
22 plants and has caused only localized effects at a few plants. It is not expected to
23 be a problem during the license renewal term.

24
25 The staff has not identified any significant new information during its independent review of
26 the McGuire ER, the staff's site visit, the scoping process, its review of monitoring programs,
27 or its evaluation of other available information. Therefore, the staff concludes that there are
28 no impacts of scouring during the renewal term beyond those discussed in the GEIS.

- 29
30 • Eutrophication. Based on information in the GEIS, the Commission found that

31
32 Eutrophication has not been found to be a problem at operating nuclear power
33 plants and is not expected to be a problem during the license renewal term.

34
35 The staff has not identified any significant new information during its independent review of
36 the McGuire ER, the staff's site visit, the scoping process, its review of monitoring programs,
37 or its evaluation of other available information. Therefore, the staff concludes that there are
38 no impacts of eutrophication during the renewal term beyond those discussed in the GEIS.

- Discharge of chlorine or other biocides. Based on information in the GEIS, the Commission found that

Effects are not a concern among regulatory and resource agencies and are not expected to be a problem during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, its evaluation of other available information including the National Pollutant Discharge Elimination System (NPDES) permit for McGuire or discussion with the NPDES compliance office. Therefore, the staff concludes that there are no impacts of discharge of chlorine or other biocides during the renewal term beyond those discussed in the GEIS.

- Discharge of sanitary wastes and minor chemical spills. Based on information in the GEIS, the Commission found that

Effects are readily controlled through NPDES permit and periodic modifications, if needed, and are not expected to be a problem during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, its evaluation of other available information including the NPDES permit for McGuire or discussion with NPDES compliance office. Therefore, the staff concludes that there are no impacts of discharges of sanitary wastes and minor chemical spills during the renewal term beyond those discussed in the GEIS.

- Discharge of other metals in wastewater. Based on information in the GEIS, the Commission found that

These discharges have not been found to be a problem at operating nuclear power plants with cooling-tower-based heat dissipation systems and have been satisfactorily mitigated at other plants. They are not expected to be a problem during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, its evaluation of other available information including the NPDES permit for McGuire or discussion with NPDES compliance office. Therefore, the staff concludes that there are no impacts of discharges of other metals in wastewater during the renewal term beyond those discussed in the GEIS.

Environmental Impacts of Operation

- Water-use conflicts (plants with once-through cooling systems). Based on information in the GEIS, the Commission found that

These conflicts have not been found to be a problem at operating nuclear power plants with once-through heat dissipation systems.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no water-use conflicts during the renewal term beyond those discussed in the GEIS.

- Accumulation of contaminants in sediments or biota. Based on information in the GEIS, the Commission found that

Accumulation of contaminants has been a concern at a few nuclear power plants but has been satisfactorily mitigated by replacing copper alloy condenser tubes with those of another metal. It is not expected to be a problem during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of available information. Therefore, the staff concludes that there are no impacts of accumulation of contaminants in sediments or biota during the renewal term beyond those discussed in the GEIS.

- Entrainment of phytoplankton and zooplankton. Based on information in the GEIS, the Commission found that

Entrainment of phytoplankton and zooplankton has not been found to be a problem at operating nuclear power plants and is not expected to be a problem during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, its review of monitoring programs, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of entrainment of phytoplankton and zooplankton during the renewal term beyond those discussed in the GEIS.

- Cold shock. Based on information in the GEIS, the Commission found that

Cold shock has been satisfactorily mitigated at operating nuclear plants with once-through cooling systems, has not endangered fish populations or been found to be a problem at operating nuclear power plants with cooling towers or cooling ponds, and is not expected to be a problem during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of cold shock during the renewal term beyond those discussed in the GEIS.

- Thermal plume barrier to migrating fish. Based on information in the GEIS, the Commission found that

Thermal plumes have not been found to be a problem at operating nuclear power plants and are not expected to be a problem during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of thermal plumes to migrating fish during the renewal term beyond those discussed in the GEIS.

- Distribution of aquatic organisms. Based on information in the GEIS, the Commission found that

Thermal discharge may have localized effects but is not expected to effect the larger geographical distribution of aquatic organisms.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, its review of monitoring programs, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts on the distribution of aquatic organisms during the renewal term beyond those discussed in the GEIS.

Environmental Impacts of Operation

- 1 • Premature emergence of aquatic insects. Based on information in the GEIS, the
2 Commission found that

3
4 Premature emergence has been found to be a localized effect at some operating
5 nuclear power plants but has not been a problem and is not expected to be a
6 problem during the license renewal term.

7
8 The staff has not identified any significant new information during its independent review of
9 the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available
10 information. Therefore, the staff concludes that there are no impacts of premature
11 emergence during the renewal term beyond those discussed in the GEIS.

- 12
13 • Gas supersaturation (gas bubble disease). Based on information in the GEIS, the
14 Commission found that

15
16 Gas supersaturation was a concern at a small number of operating nuclear
17 power plants with once-through cooling systems but has been satisfactorily
18 mitigated. It has not been found to be a problem at operating nuclear power
19 plants with cooling towers or cooling ponds and is not expected to be a problem
20 during the license renewal term.

21
22 The staff has not identified any significant new information during its independent review of
23 the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available
24 information. Therefore, the staff concludes that there are no impacts of gas supersaturation
25 during the renewal term beyond those discussed in the GEIS.

- 26
27 • Low dissolved oxygen in the discharge. Based on information in the GEIS, the
28 Commission found that

29
30 Low dissolved oxygen has been a concern at one nuclear power plant with a
31 once-through cooling system but has been effectively mitigated. It has not been
32 found to be a problem at operating nuclear power plants with cooling towers or
33 cooling ponds and is not expected to be a problem during the license renewal
34 term.

35
36 The staff has not identified any significant new information during its independent review of
37 the McGuire ER, the staff's site visit, the scoping process, its review of monitoring programs,
38 or its evaluation of other available information. Therefore, the staff concludes that there are

no impacts of low dissolved oxygen during the renewal term beyond those discussed in the GEIS.

- Losses from predation, parasitism, and disease among organisms exposed to sublethal stresses. Based on information in the GEIS, the Commission found that

These types of losses have not been found to be a problem at operating nuclear power plants and are not expected to be a problem during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of losses from predation, parasitism, and disease among organisms exposed to sub-lethal stresses during the renewal term beyond those discussed in the GEIS.

- Stimulation of nuisance organisms. Based on information in the GEIS, the Commission found that

Stimulation of nuisance organisms has been satisfactorily mitigated at the single nuclear power plant with a once-through cooling system where previously it was a problem. It has not been found to be a problem at operating nuclear power plants with cooling towers or cooling ponds and is not expected to be a problem during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of stimulation of nuisance organisms during the renewal term beyond those discussed in the GEIS.

- Microbiological organisms (occupational health). Based on information in the GEIS, the commission found that

Occupational health impacts are expected to be controlled by continued application of accepted industrial hygiene practices to minimize worker exposure.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's onsite visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there is no impacts of microbiological organisms during the renewal term beyond those discussed in the GEIS.

Environmental Impacts of Operation

- Noise. Based on information in the GEIS, the Commission found that

Noise has not been found to be a problem at operating plants and is not expected to be a problem at any plant during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of noise during the renewal term beyond those discussed in the GEIS.

The Category 2 issues related to cooling system operation during the renewal term that are applicable to McGuire are discussed in the section that follows, and are listed in Table 4-2.

4.1.1 Entrainment of Fish and Shellfish in Early Life Stages

For plants with once-through cooling systems, entrainment of fish and shellfish in early life stages into cooling water systems associated with nuclear power plants is considered a Category 2 issue, requiring a site-specific assessment prior to license renewal.

Table 4-2. Category 2 Issues Applicable to Operation of the McGuire Cooling System During the Renewal Term

ISSUE—10 CFR Part 51, Subpart A, Appendix B, Table B-1	GEIS Sections	10 CFR 51.53(c)(3)(ii) Subparagraph	SEIS Section
AQUATIC ECOLOGY (FOR PLANTS WITH ONCE-THROUGH COOLING POND HEAT-DISSIPATION SYSTEMS)			
Entrainment of fish and shellfish in early life stages	4.2.2.1.2; 4.4.3	B	4.1.1
Impingement of fish and shellfish	4.2.2.1.3; 4.4.3	B	4.1.2
Heat shock	4.2.2.1.4; 4.4.3	B	4.1.3
HUMAN HEALTH			
Microbiological organisms (public health)(plants using lakes or canals, or cooling towers or cooling ponds that discharge into a small river)	4.3.6	G	4.1.4

The staff independently reviewed the McGuire ER (Duke 2001a), visited the site, and reviewed the application for NPDES Permit No. NC0024392, which was issued by the North Carolina Department of Environment and Natural Resources (NCDENR) and expires February 28, 2005.

In response to requirements set by the North Carolina Department of Natural Resources and Community Development (NCDNRCD), Division of Environmental Management, Duke submitted a Clean Water Act (CWA) Section 316(b) demonstration for McGuire in October 1978 (Duke Power Company 1978).

The 316(b) study conclusions related to entrainment of juvenile fish were based on determinations of larval fish species composition and abundance evaluated on a biweekly basis when larval fish were present between 1974 and 1977 (Duke Power Company 1978). Species known to spawn in the McGuire intake cove are the introduced forage fish—threadfin shad (*Dorosoma petenense*), yellow perch (*Perca flavescens*), bluegill sunfish (*Lepomis macrochirus*), and crappie (*Poxomis* spp). The collection site was in the upper intake area, at a depth of 15 m (49 ft). Ichthyoplankton losses to entrainment were primarily threadfin shad eggs and larvae. Because of the rapid threadfin shad reproduction rate and the presence of more suitable spawning habitat outside the influence of the intake structures, losses do not have a measurable effect on the standing crop of shad. Most fish species that reside in the vicinity of McGuire spawn in shallow shoreline areas and produce demersal, adhesive eggs that would not be subject to entrainment. In addition, during summer up to 45 percent of the intake water was predicted to come from the low-level intake, which pulls water from the hypolimnion at a depth of approximately 30 m (100 ft). Because there are few plankton of any sort in this cold, low-oxygen water, opportunities for larval fish entrainment were expected to be further reduced during the summer period.

After reviewing Duke's submittal, the NCDNRCD concurred with the conclusions of the study (NCDNRCD 1984) and re-issued the site's NPDES permit (dated September 1, 1984) with no additional monitoring or studies required.

The staff reviewed the available information, the results of entrainment studies, and operating history of the intake and concludes that the potential impacts of the cooling-water-intake system's entrainment of fish and shellfish in the early life stages are SMALL, and additional mitigation is not warranted.

4.1.2 Impingement of Fish and Shellfish

For plants with once-through cooling systems, impingement of fish and shellfish on debris screens of cooling water systems associated with nuclear power plants is considered a Category 2 issue, requiring a site-specific assessment prior to license renewal.

Environmental Impacts of Operation

1 The staff independently reviewed the McGuire ER (Duke 2001a), visited the site, and reviewed
2 the application for NPDES Permit No. NC0024392, which was issued by the NCDENR and
3 expires February 28, 2005.

4
5 In response to requirements set by the North Carolina Department of Natural Resources and
6 Community Development (NCDNRCD), Division of Environmental Management, Duke submitted
7 a Clean Water Act (CWA) Section 316(b) demonstration for McGuire in October 1978 (Duke
8 Power Company 1978).

9
10 The 316(b) study conclusions related to impingement of fish and shellfish were based on studies
11 of fish species composition and abundance evaluated on a monthly, quarterly, or annual basis
12 using electrofishing, gillnetting, and rotenone sampling techniques between 1974 and 1977
13 (Duke Power Company 1978). Based on studies conducted in the 1970s, most fish impinged at
14 McGuire were threadfin shad, especially during the fall and winter when the introduced species
15 is susceptible to low-temperature stress and exhibits high mortality associated with cool water
16 temperatures. Fish swimming between the trash racks and screens were predicted to be most
17 susceptible to impingement. However, it was predicted that fish approaching the upper-level
18 trash racks when the low-level pumps were operating could be repelled by the low temperature
19 and oxygen levels associated with water drawn from the hypolimnion by the low-level pumps.

20
21 After reviewing Duke's submittal, the NCDNRCD concurred with the conclusions of the study
22 (NCDNRCD 1984) and re-issued the site's NPDES permit (dated September 1, 1984), with no
23 additional monitoring or studies required.

24
25 An in-house impingement sampling program that began in December 2000 and is scheduled to
26 continue through November 2002 incorporates a full count of all fishes impinged on condenser
27 cooling water intake screens for Units 1 and 2 through a weekly sampling program (Duke
28 2001b). Preliminary results indicate that impingement rates at McGuire are very low. Between
29 December 2000 and December 2001, a total of 1746 fish were impinged. Weekly impingement
30 ranged from a low of 5 fish to a high of 455 fish. Threadfin shad was the species most
31 commonly impinged (50 percent). Seventy-one percent of these threadfin shad were observed
32 during a 14-day period between December 29, 2000, and January 12, 2001, when the water
33 temperature was a low 10°C. Threadfin shad are a nonindigenous, temperate species with
34 documented potential for cold shock morbidity and mortality when water temperatures drop
35 below 9°C (Strawn 1963). These data suggest that the high impingement rate for threadfin
36 shad during the 14-day period resulted from a natural die-off in the vicinity of the intake. Other
37 species observed on the intake screens were bluegill sunfish (*Lepomis macrochirus*; 9 percent),
38 alewife (*Alosa pseudoharengus*; 8 percent), and a combination of other species that individually
39 comprised less than 5 percent of the total number impinged (30 percent).

Impacts to shellfish from impingement are not considered important because adult shellfish are not motile and susceptible to impingement.

The staff has reviewed the available information relative to potential impacts of the cooling water intake on the impingement of fish and shellfish and, based on this data, concludes that the impacts are SMALL, and additional mitigation is not warranted.

4.1.3 Heat Shock

For plants with once-through cooling systems, the effects of heat shock are listed as a Category 2 issue and require plant-specific evaluation before license renewal.

The staff independently reviewed the McGuire ER (Duke 2001a), visited the site, and reviewed the application for NPDES Permit No. NC0024392, which was issued by the NCDENR and expires February 28, 2005.

Duke submitted a CWA Section 316(a) demonstration for McGuire to the NCDNRCD, Division of Environmental Management, in June 1985 (Duke 1985). In summary, the NCDNRCD indicated that "the effects of the discharge from the McGuire Nuclear Station is such that the protection and propagation of a balanced indigenous aquatic community is assured in Lake Norman and that interaction of the two thermal plumes of McGuire and Marshall do not occur" (NCDNRCD 1985). Thus, the 316(a) submittal was successful and suggested that the limits in the NPDES permit were sufficient to protect the aquatic environment of Lake Norman.

Studies performed for the 316(a) submittal were initiated in 1973 and continued through submission of the document. Physical and mathematical models were developed to determine Lake Norman hydrodynamics and thermal plume characteristics in relation to station operation (Duke Power Company 1985). Both models were validated with surface-temperature data and were found to predict surface thermal plume size with a high degree of confidence. Both predicted that operation of McGuire would not result in discharge temperatures outside those allowed in the NPDES permit. Fish species collected during preoperational and operational studies indicated no substantial change in species composition over time (Duke Power Company 1985). The most significant changes were increases in specific fish taxa abundance in winter at the McGuire discharge, associated with fish congregating in the discharge plume due to increased water temperature.

McGuire currently operates under thermal limits established in its NPDES permit issued February 1, 1990. Annual aquatic monitoring to assess impacts of current thermal limits on the aquatic biota of Lake Norman is required. Results of the monitoring studies conducted in support of this requirement are reported annually to the NCDENR (formally NCDNRCD).

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Monitoring of fish populations in and around the McGuire mixing zone is coordinated with the North Carolina Wildlife Resource Commission (NCWRC). The latest report covers data collected in 1999 (Duke 2000). Observed striped bass mortalities during the summer of 1999 included one mortality within the mixing zone and five mortalities in the main channel outside the mixing zone which may or may not have been related to heat shock. Shoreline electrofishing catches at the McGuire mixing zone area were only slightly lower than a reference area in total biomass and taxa composition. Hydroacoustic and purse seine sampling were also conducted in 1999, in cooperation with the NCWRC, to evaluate Lake Norman forage fish populations. According to the applicant, "fisheries data to date indicate that the Lake Norman fishery is consistent with the trophic status and productivity of the reservoir" (Duke 2000).

Based on its review of available information, the staff concludes that the potential heat shock impacts resulting from operation of the plant's cooling water discharge system to the aquatic environment on or in the vicinity of the site are SMALL, and additional mitigation is not warranted.

4.1.4 Microbiological Organisms (Public Health)

McGuire has a once-through cooling system that uses the Catawba River as the cooling source. The Catawba River, which was impounded from Lake Norman, has an annual average flow rate of $2.38\text{E}+9$ cubic meters per year ($8.42\text{E}+10$ cubic feet per year). This flow rate is lower than the $9\text{E}+10$ cubic meters per year ($3.15\text{E}+12$ cubic feet per year) 10 CFR 51.53 (c)(3)(ii)(G), which requires an evaluation of potentially harmful thermophylic (heat-loving) microorganisms on human health. The flow rate raises a concern from the standpoint of the potential for enhancement of thermophylic microorganisms such as *Naegleria fowleri*. This type of organism could be a potential health concern for members of the public swimming in the cooling source and can under certain conditions cause a fatal condition called primary amebic meningoencephalitis.

Lake Norman is a popular site for a variety of water-based recreational activities, including boating, fishing, water skiing, and swimming. All of these activities are dispersed throughout the lake, rather than being concentrated in certain areas. Swimming occurs from private boat docks and piers located around the lake shoreline and from boats anchored offshore.

The Catawba River, which was impounded to form Lake Norman, has an annual average flow rate of $75.6\text{ m}^3/\text{s}$ (2670 cubic feet per second). McGuire uses Lake Norman as a source for condenser cooling water. The heated effluent from the condenser discharge enters Lake Norman through a discharge canal. This canal is 1 km (0.6 mi) long with an average depth of 12.2 m (40 ft). The heated effluent mixes initially in the canal with surface waters of the main

lake before stabilizing vertically and spreading over the lake surface, ultimately dissipating its heat to the atmosphere.

No swimming or boating is allowed in the canal, although fishing is permitted from its banks. Boating, fishing, and water contact activities take place at the confluence of the canal and the lake. The closest privately owned dock is located outside the 760-m (2500-ft) exclusion zone and is approximately 150 m (495 ft) from the confluence of the canal and the lake.

The state agency responsible for public health is the North Carolina Department of Health and Human Services (NCDHHS), Division of Public Health. Duke consulted with this agency as to whether there is a concern about the potential existence and concentration of *N. fowleri* in the receiving waters for the plant cooling discharge waters. By letter dated June 12, 2000, the Division of Public Health summarized the agency's position and opinion regarding the risk to individuals using Lake Norman for recreational activities and found that only a small percentage of cases of PAME have been associated with thermally enhanced waters and the disease is exceedingly rare given the millions of swimming events in warm fresh water bodies in the United States, therefore, the NCDHHS feels the risk to individuals utilizing Lake Norman for recreational activities is extremely low.

There has been no known impact of operation of McGuire on public health related to thermophilic microorganisms and consultation with the NCDHHS. This data indicates that the impact of deleterious microbiological organisms during continued operation of the plant during the renewal term is low.

Based on its review of the above information, the staff concludes that the potential impacts to public health from microbiological organisms resulting from operation of the plant's cooling water discharge system to the aquatic environment on or in the vicinity of the site are SMALL, and additional mitigation is not warranted.

4.2 Transmission Lines

The McGuire ER (Duke 2001a) describes four transmission lines with a total length of 4.5 km (2.8 mi) that connect the McGuire plant to two substations within the local transmission system. These lines are situated on 2.2 km (1.4 mi) of corridor on approximately 22.8 ha (56.2 ac). Transmission corridor rights-of-way are generally maintained on a 3-year cycle. Mechanical mowing and selective herbicide application are the standard methods of corridor maintenance. Duke cooperates with the U.S. Fish and Wildlife Service (FWS) and North Carolina Natural Heritage Program to identify Federally and State-listed species, special habitats, new findings, and other pertinent factors. This information is used to establish new and review existing

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vegetation management programs for the rights-of-way so that adverse impacts to these may be avoided during corridor maintenance. As noted in Section 2.1.7, the NRC staff conducted a separate evaluation of the rights-of-way from the McGuire station to the Oconee Nuclear Station, in South Carolina, under the Supplemental Generic Environmental Impact Statement for Oconee Nuclear Station (NRC 1999a).

Category 1 issues in 10 CFR Part 51, Subpart A, Appendix B, Table B-1, that are applicable to the McGuire transmission lines are listed in Table 4-3. Duke stated in its ER that "no new information existed for the issues that would invalidate the GEIS conclusions" (Duke 2001a). The staff has not identified any significant new information during its independent review of the McGuire ER (Duke 2001a), the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts related to these issues beyond those discussed in the GEIS. For all of those issues, the GEIS concluded that the impacts are SMALL, and plant-specific mitigation measures are not likely to be sufficiently beneficial to be warranted.

Table 4-3. Category 1 Issues Applicable to the McGuire Nuclear Station Transmission Lines During the Renewal Term

ISSUE - 10 CFR Part 51, Subpart A, Appendix B, Table B-1	GEIS Section
Terrestrial Resources	
Power line right-of-way management (cutting and herbicide application)	4.5.6.1
Bird collisions with power lines	4.5.6.2
Impacts of electromagnetic fields on flora and fauna (plants, agricultural crops, honeybees, wildlife, and livestock)	4.5.6.3
Floodplains and wetlands on power line right-of-way	4.5.7
Air Quality	
Air quality effects of transmission lines	4.5.2
Land Use	
Onsite land use	4.5.3
Power line right-of-way	4.5.3

A brief description of the staff's review and GEIS conclusions, as codified in Table B-1, for each of these issues follows:

- Power line right-of-way management (cutting and herbicide application). Based on information in the GEIS, the Commission found that

The impacts of right-of-way maintenance on wildlife are expected to be of small significance at all sites.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, discussions with the FWS, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of power line right-of-way maintenance during the renewal term beyond those discussed in the GEIS.

- Bird collisions with power lines: Based on information in the GEIS, the Commission found that

Impacts (of bird collisions with power lines) are expected to be of small significance at all sites.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, discussions with the FWS, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of bird collisions with power lines during the renewal term beyond those discussed in the GEIS.

- Impacts of electromagnetic fields on flora and fauna (plants, agricultural crops, honeybees, wildlife, livestock): Based on information in the GEIS, the Commission found that

No significant impacts of electromagnetic fields on terrestrial flora and fauna have been identified. Such effects are not expected to be a problem during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of electromagnetic fields on flora and fauna during the renewal term beyond those discussed in the GEIS.

- Floodplains and wetlands on power line right-of-way: Based on information in the GEIS, the Commission found that

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Periodic vegetation control is necessary in forested wetlands underneath power lines and can be achieved with minimal damage to the wetland. No significant impact is expected at any nuclear power plant during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, discussions with the FWS, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts on floodplains and wetlands on the power line rights-of-way during the renewal term beyond those discussed in the GEIS.

- Air quality effects of transmission lines: Based on information in the GEIS, the Commission found that

Production of ozone and oxides of nitrogen is insignificant and does not contribute measurably to ambient levels of these gases.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no air quality impacts of transmission lines during the renewal term beyond those discussed in the GEIS.

- Onsite land use: Based on information in the GEIS, the Commission found that

Projected onsite land use changes required during ... the renewal period would be a small fraction of any nuclear power plant site and would involve land that is controlled by the applicant.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no onsite land-use impacts during the renewal term beyond those discussed in the GEIS.

- Power line right-of-way (land use). Based on information in the GEIS, the Commission found that

Ongoing use of power line right of ways would continue with no change in restrictions. The effects of these restrictions are of small significance.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts on use of power line rights-of-way during the renewal term beyond those discussed in the GEIS.

There is one Category 2 issue related to transmission lines, and another issue related to transmission lines is being treated as a Category 2 issue. These issues are listed in Table 4-4 and are discussed in Sections 4.2.1 and 4.2.2.

Table 4-4. Category 2 Issues Applicable to the McGuire Transmission Lines During the Renewal Term

ISSUE—10 CFR Part 51, Subpart A, Appendix B, Table B-1	GEIS Section	10 CFR 51.53(c)(3)(ii) Subparagraph	SEIS Section
HUMAN HEALTH			
Electromagnetic fields, acute effects (electric shock)	4.5.4.1	H	4.2.1
Electromagnetic fields, chronic effects	4.5.4.2	NA	4.2.2

4.2.1 Electromagnetic Fields—Acute Effects

In the GEIS (NRC 1996), the staff found that without a review of the conformance of each nuclear plant transmission line with *National Electrical Safety Code* (NESC) criteria, (Institute of Electrical and Electronic Engineers [IEEE] 1997) it was not possible to determine the significance of the electric shock potential. Evaluation of individual plant transmission lines is necessary because the issue of electric shock safety was not addressed in the licensing process for some plants. For other plants, land use in the vicinity of transmission lines may have changed, or power distribution companies may have chosen to upgrade line voltage. To comply with 10 CFR 51.53(c)(3)(ii)(H), the applicant must provide an assessment of the potential shock hazard if the transmission lines that were constructed specifically to connect the plant to the transmission system do not meet the recommendations of the NESC for preventing electric shock from induced currents.

Two 230-kV transmission lines and two 525-kV transmission lines connect the McGuire Nuclear Station to the transmission system. The 230-kV lines connect McGuire Unit 1 to a 230-kV switchyard and have a length of approximately 1200 m (4000 ft). Similarly, the 525-kV lines connect Unit 2 to a 525-kV switchyard and have a length of approximately 1000 m (3300 ft). The two switchyards are adjacent to each other.

1 The transmission lines were constructed to meet the 1973 NESC requirements. Duke (2001a)
2 has compared the clearances calculated using the 1973 NESC with clearance requirements of
3 the 1997 NESC and found the 1973 NESC clearance requirements to be greater. Duke further
4 states that measured clearances from the sagged plan and profile of each bus line indicate that
5 the designed clearances of the transmission lines exceed the 1997 NESC vertical clearance
6 requirements and that there have been no changes in the design voltages of the lines.
7 Therefore, the staff concludes that the impact of the potential for electric shock is SMALL, and
8 additional mitigation is not warranted.

9 10 **4.2.2 Electromagnetic Fields—Chronic Effects**

11
12 In the GEIS, the chronic effects of 60-Hz electromagnetic fields from power lines were not
13 designated as Category 1 or 2 and will not be until a scientific consensus is reached on the
14 health implications of these fields.

15
16 The potential for chronic effects from these fields continues to be studied and is not known at
17 this time. The National Institute of Environmental Health Sciences (NIEHS) directs related
18 research through the U.S. Department of Energy (DOE). A recent report (NIEHS 1999) contains
19 the following conclusion:

20
21 The NIEHS concludes that ELF-EMF [extremely low frequency-electromagnetic field]
22 exposure cannot be recognized as entirely safe because of weak scientific evidence that
23 exposure may pose a leukemia hazard. In our opinion, this finding is insufficient to
24 warrant aggressive regulatory concern. However, because virtually everyone in the
25 United States uses electricity and therefore is routinely exposed to ELF-EMF, passive
26 regulatory action is warranted such as a continued emphasis on educating both the
27 public and the regulated community on means aimed at reducing exposures. The NIEHS
28 does not believe that other cancers or non-cancer health outcomes provide sufficient
29 evidence of a risk to currently warrant concern.

30
31 This statement is not sufficient to cause the staff to change its position with respect to the
32 chronic effects of electromagnetic fields. The staff considers the GEIS finding of "not applicable"
33 still appropriate and will continue to follow developments on this issue.
34

4.3 Radiological Impacts of Normal Operations

Category 1 issues in 10 CFR Part 51, Subpart A, Appendix B, Table B-1 that are applicable to McGuire Units 1 and 2 in regard to radiological impacts are listed in Table 4-5. Duke stated in its ER (Duke 2001a) that “no new information existed for the issues that would invalidate the GEIS conclusion.” The staff has not identified any significant new information during its independent review of the McGuire ER (Duke 2001a), the staff’s site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts related to these issues beyond those discussed in the GEIS. For all of these issues, the staff concluded in the GEIS that the impacts are SMALL, and additional plant-specific mitigation measures are not likely to be sufficiently beneficial to be warranted.

Table 4-5. Category 1 Issues Applicable to Radiological Impacts of Normal Operations During the Renewal Term

ISSUE—10 CFR Part 51, Subpart A, Appendix B, Table B-1	GEIS Section
HUMAN HEALTH	
Radiation exposures to public (license renewal term)	4.6.2
Occupational radiation exposures (license renewal term)	4.6.3

A brief description of the staff’s review and the GEIS conclusions, as codified in Table B-1, for each of these issues follows:

- Radiation exposures to public (license renewal term). Based on information in the GEIS, the Commission found that

Radiation doses to the public will continue at current levels associated with normal operations.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff’s site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of radiation exposures to the public during the renewal term beyond those discussed in the GEIS.

- Occupational radiation exposures (license renewal term). Based on information in the GEIS, the Commission found that

Projected maximum occupational doses during the license renewal term are within the range of doses experienced during normal operations and normal maintenance outages, and would be well below regulatory limits.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of occupational radiation exposures during the renewal term beyond those discussed in the GEIS.

There are no Category 2 issues related to radiological impacts of routine operations.

4.4 Socioeconomic Impacts of Plant Operations During the License Renewal Period

Category 1 issues in 10 CFR Part 51, Subpart A, Appendix B, Table B-1 that are applicable to socioeconomic impacts during the renewal term are listed in Table 4-6. Duke stated in its ER (Duke 2001a) that "no new information existed for the issues that would invalidate the GEIS conclusions." The staff has not identified any significant new information during its independent review of the McGuire ER (Duke 2001a), the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts related to these issues beyond those discussed in the GEIS (NRC 1996). For all of those issues, the staff concluded in the GEIS that the impacts are SMALL, and plant-specific mitigation measures are not likely to be sufficiently beneficial to be warranted.

Table 4-6. Category 1 Issues Applicable to Socioeconomics During the Renewal Term

ISSUE—10 CFR Part 51, Subpart A, Appendix B, Table B-1	GEIS Sections
SOCIOECONOMICS	
Public services: public safety, social services, and tourism and recreation	4.7.3; 4.7.3.3; 4.7.3.4; 4.7.3.6
Public services: education (license renewal term)	4.7.3.1
Aesthetic impacts (license renewal term)	4.7.6
Aesthetic impacts of transmission lines (license renewal term)	4.5.8

A brief description of the staff's review and the GEIS conclusions, as codified in Table B-1, for each of these issues follows:

- Public services—public safety, social services, and tourism and recreation. Based on information in the GEIS, the Commission found that

Impacts to public safety, social services, and tourism and recreation are expected to be of small significance at all sites.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts on public safety, social services, and tourism and recreation during the renewal term beyond those discussed in the GEIS.

- Public services—education (license renewal term). Based on information in the GEIS, the Commission found that

Only impacts of small significance are expected.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts on education during the renewal term beyond those discussed in the GEIS.

- Aesthetic impacts (license renewal term). Based on information in the GEIS, the Commission found that

No significant impacts are expected during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no aesthetic impacts during the renewal term beyond those discussed in the GEIS.

- Aesthetic impacts of transmission lines (license renewal term). Based on information in the GEIS, the Commission found that

No significant impacts are expected during the license renewal term.

The staff has not identified any significant new information during its independent review of the McGuire ER, the staff's site visit, the scoping process, or its evaluation of other available

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information. Therefore, the staff concludes that there are no aesthetic impacts of transmission lines during the renewal term beyond those discussed in the GEIS.

Table 4-7 lists the Category 2 socioeconomic issues, which require plant-specific analysis, and environmental justice, which was not addressed in the GEIS.

Table 4-7. Environmental Justice Analysis and GEIS Category 2 Issues Applicable to Socioeconomics During the License Renewal Term

ISSUE—10 CFR Part 51, Subpart A, Appendix B, Table B-1	GEIS Section	10 CFR 51.53(c)(3)(ii) Subparagraph	SEIS Section
SOCIOECONOMICS			
Housing impacts	4.7.1	I	4.4.1
Public services: public utilities	4.7.3.5	I	4.4.2
Offsite land use (license renewal term)	4.7.4	I	4.4.3
Public Services, transportation	4.7.3.2	J	4.4.4
Historic and archaeological resources	4.7.7	K	4.4.5
Environmental Justice	Not Addressed ^(a)	Not Addressed ^(a)	4.4.6

(a) Guidance related to environmental justice was not in place at the time the GEIS and the associated revision to 10 CFR Part 51 were prepared. Therefore, environmental justice are to be addressed in the licensee's ER and the staff's EIS.

4.4.1 Housing Impacts During Operations

10 CFR Part 51, Subpart A, Appendix B, Table B-1 states that impacts on housing availability are expected to be of small significance at plants located in a high-population area where growth-control measures are not in effect. SMALL impacts result when no discernible change in housing availability occurs, changes in rental rates and housing values are similar to those occurring statewide, and no housing construction or conversion is required to meet new demand (NRC 1996). Increases in rental rates or housing values in these areas would be expected to equal or slightly exceed the statewide inflation rate. No extraordinary construction or conversion of housing would occur where small impacts are foreseen.

The impacts on housing are considered to be of MODERATE significance when there is a discernible but short-lived reduction in available housing units because of project-induced in-migration. The impacts on housing are considered to be of LARGE significance when project-related demand for housing units would result in very limited housing availability and

would increase rental rates and housing values well above normal inflationary increases in the state. MODERATE and LARGE impacts are possible at sites located in rural and remote areas, at sites located in areas that have experienced extremely slow population growth (and thus slow or no growth in housing), or where growth control measures that limit housing development are in existence or have been recently lifted. Because impact significance depends on local conditions, housing is a Category 2 issue (NRC 1996).

The NRC has developed a method of characterizing population that is based on two factors: "sparseness" and "proximity" (NRC 1996). "Sparseness" measures population density and city size within 32-km (20-mi) of the site. "Proximity" measures population density and city size within 80 km (50 mi). In these calculations, the density is averaged over the land area covered by the ring; large water bodies are excluded. Each factor has categories of density and city size and a matrix is used to rank the population category as low, medium or high.

An analysis of the 2000 census data indicates that 781,783 people live within a 32-km (20-mi) radius of McGuire with an average population density of 240 persons/km² (622 persons/mi²). There are also four communities of 25,000 or more in this area (Table 4-8). This population density and number of cities correspond to "sparseness" Category 4, "least sparse." An analysis of the 2000 census data also indicates that 2,309,976 people live within 80 km (50 mi) of McGuire, with an average population density of 114 persons/km² (294 persons/mi²). There is one city, Charlotte, with a population of 100,000 or more in this area. This population density and number of cities correspond to "proximity" Category 4 "in close proximity." According to the GEIS, these "sparseness" and "proximity" sources indicate that McGuire is located in a high-population area.

Table 4-8. Analysis of Population "Sparseness" and "Proximity" in the Vicinity of McGuire

Radial Distance from McGuire	2000 Census Population	Population Density persons/km ² (persons/mi ²)	Communities of 25,000 or More Persons	Cities of 100,000 or More Persons
32 km (20 mi)	781,783	240 (622)	3	1
80 km (50 mi)	2,309,976	114 (294)	6	1

McGuire is located in northwestern Mecklenburg County, approximately 27 km (17 mi) north-northwest of Charlotte, North Carolina, within the rapidly developing Charlotte metropolitan area. There are no prohibitions on the development of residential housing within Iredell, Mecklenburg, Gaston, or Lincoln counties. In the McGuire ER, Duke made the case for considering no further employment increases for its operating Units 1 and 2 rather than the standard GEIS assumption of 60 new employees per unit (Duke 2001a). Adding full-time

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employees to the plant workforce for the license renewal operating term would have the potential indirect effect of creating additional jobs and related population growth in the community. Section 4.14.2 of Supplement 1 to Regulatory Guide 4.2 (NRC 2000) states: "If additional workers are not anticipated there will be no impact on housing and no further analysis is required." McGuire has approximately 1370 full-time workers employed by Duke or site contractors during normal plant operations. Duke does not anticipate that additional full-time workers will be employed during the license renewal period. Therefore, no analysis is required for this issue.

Duke has concluded that the impact on housing from the continued operation of McGuire will be SMALL and that no mitigation is required. This conclusion is based on the following:

- (1) Duke does not anticipate an increase in employment during the license renewal period.
- (2) The number of McGuire employees will continue to be a small percentage of the population in the adjacent counties during the period of the extended license.

The staff reviewed the available information relative to housing impacts and Duke's conclusions. Based on this review, the staff concludes that the impact on housing during the license renewal period would be SMALL, and additional mitigation is not warranted.

4.4.2 Public Services: Public Utility Impacts During Operations

Impacts on public utility services are considered SMALL if there is little or no change in the ability of the system to respond to the level of demand, and thus there is no need to add capital facilities. Impacts are considered MODERATE if overtaxing of service capabilities occurs during periods of peak demand. Impacts are considered LARGE if existing levels of service (e.g., water or sewer services) are substantially degraded and additional capacity is needed to meet ongoing demands for services. In the GEIS, the staff concluded that, in the absence of new and significant information to the contrary, the only impacts on public utilities that could be significant are impacts on public water supplies (NRC 1996).

There are no identified increases in demand of the water supplied by the Charlotte-Mecklenburg Utilities District (CMUD) during the period of extended operation at McGuire. The current water use at McGuire, from water supplied by CMUD, is 0.03 percent of the average daily demand on the CMUD system. Duke does not anticipate that additional workers will be employed during the period of extended operations. Therefore, there will be no impact to public utilities from additional plant workers.

The staff reviewed the available information relative to public utility services impacts and Duke's conclusions. Based on this review, the staff concludes that the impact on public utilities during the license renewal period would be SMALL, and additional mitigation is not warranted.

4.4.3 Offsite Land Use During Operations

Offsite land use during the license renewal term is a Category 2 issue (10 CFR 51, Subpart A, Appendix B, Table B-1). Table B-1 of 10 CFR 51 Subpart A, Appendix B notes that "significant changes in land use may be associated with population and tax revenue changes resulting from license renewal."

The GEIS (NRC 1996) defines the magnitude of land-use changes as a result of plant operation during the license renewal term as follows:

SMALL – Little new development and minimal changes to an area's land-use pattern.

MODERATE – Considerable new development and some changes to the land-use pattern.

LARGE – Large-scale new development and major changes in the land-use pattern.

Based on predictions for the case study plants, the staff projected that all new population-driven land-use changes during the license renewal term at all nuclear plants will be small because population growth caused by license renewal will represent a much smaller percentage of the local area's total population than has operations-related growth. Also, any conflicts between offsite land use and nuclear plant operations are expected to be small (NRC 1996).

Duke concluded (Duke 2001a) that there will be no adverse impact to the offsite land use from plant related population growth because they do not anticipate that additional workers will be employed at McGuire during the period of extended operations.

Tax revenue can affect land use because it enables local jurisdictions to be able to provide the public services (e.g., transportation and utilities) necessary to support development. In the GEIS, the staff states that the assessment of tax-driven land-use impacts during the license renewal term should consider (1) the size of the plant's payments relative to the community's total revenues, (2) the nature of the community's existing land-use pattern, and (3) the extent to which the community already has public services in place to support and guide development (NRC 1996).

In general, if a plant's tax payments are projected to be small relative to the community's total revenue, new tax-driven land-use changes during the plant's license renewal term would be

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SMALL. If the plant's tax payments are projected to be medium to large relative to the community's total revenue, new tax-driven land-use charges would be MODERATE. If the plant's tax payments are projected to be a dominant source of the community's total revenue, new tax-driven land-use changes would be LARGE.

In the GEIS, the staff states that if tax payments by the plant owner are less than 10 percent of the taxing jurisdictions revenue, the significance level would be SMALL, MODERATE if the plant tax payments represent 10 to 20 percent, and LARGE if the payments are over 20 percent of the jurisdiction's revenues.

The payments made by McGuire represented 7 percent of the property tax revenues and 4 percent of the total revenues collected by the town of Huntersville; the percentages are 2 percent and 1 percent for Mecklenburg County (Table 2.11). No major refurbishment activities are anticipated during the period of license renewal at McGuire. The relative importance of tax payments to Mecklenburg County would slowly decline as other development occurs.

The impacts from tax driven offsite land-use changes will be SMALL for the following reasons:

- (1) The significance of tax payments made by Duke for McGuire to local governments will be continue to be SMALL.
- (2) The area around McGuire has pre-established land patterns of development, such as land use plans and controls. McGuire is located within the town of Huntersville's planning zone.
- (3) The area around McGuire has public services in place to support and guide development. Therefore, the impact to tax-driven land-use changes from the continued payment of property taxes at McGuire is SMALL and no mitigation is required.

The staff reviewed the available information relative to land use impacts and Duke's conclusions. Based on this review, the staff concludes that the impact on land use during the license renewal period would be SMALL, and additional mitigation is not warranted.

4.4.4 Public Services: Transportation Impacts During Operations

On October 4, 1999, 10 CFR 51.53(c)(3)(ii)(J) and 10 CFR Part 51, Subpart A, Appendix B, Table B-1 were revised to clearly state that "Public Services: Transportation Impacts During Operations" is a Category 2 issue (see NRC 1999 for more discussion of this clarification). The issue is treated as such in this Supplemental Environmental Impact Statement (SEIS).

Approximately 1345 workers are currently employed at McGuire during normal plant operations (non-outage periods). The workers employed at McGuire reside primarily in Mecklenburg County and in adjoining counties. An average of 1015 additional workers are onsite during plant outage periods. The plant outages last from 30 to 40 days and occur about every 18 to 24 months. There are no identified increases in the total number of employees that will be onsite during the term of the renewed license. As shown in Table 2-3, the workers employed at McGuire reside in locations that are well distributed geographically. Therefore, with the exception of travel along North Carolina Highway 73 (NC-73), the workers would travel to the plant along many different routes.

The North Carolina Department of Transportation classifies some of the segments of NC-73 in the vicinity of McGuire as having Level of Service (LOS) D. This is a regional growth and transportation planning issue. However, Duke has taken the following steps to minimize the impacts to local traffic:

- (1) The starting times for workers at the station has been staggered to minimize the impact of plant workers entering and leaving the site.
- (2) Workers leaving the site and traveling east on NC-73 are requested to use the east entrance, and those workers traveling west on NC-73 are requested to use the west entrance.
- (3) Turn lanes have been added on NC-73 for plant traffic. Traveling east to west on NC-73, there are right turn lanes into the plant site at both entrances. Traveling west to east on NC-73, there is a left turn lane at the east plant entrance.

There are no identified increases in the total number of employees that will be onsite during the term of the renewed license. Increases in traffic capacity will be required to accommodate the projected growth in the population in the areas adjacent to McGuire. The growth in population in the area near McGuire will not be attributed to increases in employment at McGuire. Therefore, the impact of continued operation of McGuire on any future degradation in traffic service will be SMALL, and no mitigation measures are warranted.

The staff reviewed Duke's assumptions and resulting conclusions. The staff concludes that any impact of McGuire on transportation service degradation is likely to be SMALL and would not require additional mitigation.

4.4.5 Historic and Archaeological Resources

The National Historic Preservation Act (NHPA) requires that Federal agencies take into account the effects of their undertakings on historic properties. The historic preservation review process mandated by Section 106 of the NHPA is outlined in regulations issued by the Advisory Council on Historic Preservation at 36 CFR Part 800. Renewal of an operating license (OL) is an undertaking that could potentially affect historic properties. Therefore, according to the NHPA, the NRC is to make a reasonable effort to identify historic properties in the areas of potential effects. If no historic properties are present or affected, NRC is required to notify the State Historic Preservation Officer (SHPO) before proceeding. If it is determined that historic properties are present, the NRC is required to assess the possible adverse effects of the undertaking.

On January 26, 2000, Duke wrote to the North Carolina SHPO, requesting its comment on the McGuire license renewal process and on the determination by Duke that the continued operation of McGuire will have no effect on historic properties (Huff 2000). In a response dated January 31, 2000, the North Carolina SHPO stated that the extension of the operating license was not an undertaking that is likely to affect historic properties; thus, no further compliance with Section 106 was required (Brook 2000).

Due to disturbance by historic agriculture and the original construction of McGuire, it is unlikely that significant historic resources are present on the McGuire site. Major refurbishment of McGuire is not required during the license renewal period, and it is anticipated that there will be no need to utilize the few currently undeveloped portions of McGuire for operations during the renewal period. Continued operation of McGuire would have a beneficial effect on any potential unknown or undiscovered historic or archaeological resources in undisturbed areas for the duration of the license renewal period by protecting the natural landscape and vegetation and by providing restricted access to the plant.

However, care should be taken by the licensee while undertaking normal operational and maintenance activities to ensure that historic properties are not inadvertently impacted. These activities may include not only operation of the plant itself, but also land management-related actions such as recreation, wildlife habitat enhancement, or maintaining/upgrading plant access roads through the plant site.

Based on the staff's cultural resources analysis and consultation, the staff concludes that the potential impacts on historic and archaeological resources are SMALL, and no additional mitigation is warranted.

4.4.6 Environmental Justice

Environmental justice refers to a Federal policy that requires that Federal agencies identify and address, as appropriate, disproportionately high and adverse human health or environmental effects of its actions on minority^(a) or low-income populations. The memorandum accompanying Executive Order 12898 (59 FR 7629) directs Federal executive agencies to consider environmental justice under the National Environmental Policy Act of 1969 (NEPA). The Council on Environmental Quality (CEQ) has provided guidance for addressing environmental justice (CEQ 1997). Although the Executive Order is not mandatory for independent agencies, the NRC has voluntarily committed to undertake environmental justice reviews. Specific guidance is provided in NRC Office of Nuclear Reactor Regulation (NRR) Office Instruction LIC-203, "Procedural Guidance for Preparing Environmental Assessments and Considering Environmental Issues" (NRC 2001).

The environmental justice review involves identifying offsite environmental impacts, their geographic locations, minority and low-income populations that may be affected, the significance of such effects, and whether they are disproportionately high and adverse compared to the population at large within the geographic area, and if so, what mitigative measures are available and which will be implemented.

For the staff's review, a minority population is defined to exist if the percentage of each minority or aggregated minority category within the census block groups^(b) potentially affected by renewal of the McGuire OLs exceeds the corresponding percentage of minorities in a comparison area, by convention, the state, by 20 percentage points, or if the corresponding percentage of minorities within the census block group is at least 50 percent. A low-income population is defined to exist if the percentage of low-income population within a census block group exceeds the corresponding percentage of low-income population in a comparison area, by convention the state, by 20 percentage points, or if the corresponding percentage of low-income population

(a) The NRC guidance for performing environmental justice reviews defines "minority" as American Indian or Alaskan Native; Asian; Native Hawaiian or other Pacific Islander; or Black races; or Hispanic ethnicity. "Other" races and multi-racial individuals may be considered a separate minority category as well as multi-racial individuals (NRC 2001).

(b) A census block group is a combination of census blocks, which are statistical subdivisions of a census tract. A census block is the smallest geographic entity of which the Census Bureau collects and tabulates decennial census information. A census tract is a small, relatively permanent statistical subdivision of counties delineated by local committees of census data users in accordance with Census Bureau guidelines for the purpose of collecting and presenting decennial census data. Census block groups are subsets of census tracts.

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1 within a census block group is at least 50 percent. For McGuire, the minority and low-income
2 status were calculated by comparing the census block groups with the states in which, they are
3 located i.e., North Carolina and South Carolina.

4
5 Within a 80 km (50-mi) radius of McGuire, 24.5 percent of the population are minorities. Also
6 within that 80 km (50-mi) radius, 284 block groups with minority populations meet the definition
7 outlined in the NRC review guidance (NRC 2001). This represents 11.5 percent of the total
8 number of block groups within the 80-km (50-mi) radius. These populations are shown in
9 Figure 4-1. The majority of these block groups are located in urban areas associated with
10 Charlotte, Gastonia, Statesville, and Salisbury, North Carolina and Rock Hill, South Carolina.
11 There are no known environmental pathways by which these minority populations would be
12 disproportionately and adversely affected by the renewal of the McGuire license.

13
14 Low-income households comprise 11 percent of all households located within a 80-km (50-mi)
15 radius of McGuire. Within the 80-km (50-mi) radius, there are 88 low-income block groups. This
16 represents 5.5 percent of the total number of block groups within the 80-km (50-mi) radius.
17 These populations are shown in Figure 4-2.^(a) The majority of these block groups are located in
18 the urban areas of Charlotte and Gastonia, North Carolina, and Gaffney, South Carolina. There
19 are no known environmental pathways by which these low-income populations would be
20 disproportionately and adversely affected by the renewal of the McGuire license.

21
22 As part of its environmental assessment of this proposed action, Duke has determined that no
23 significant offsite environmental impacts will be created by the renewal of the McGuire OLs.
24 This conclusion is supported by the review performed of the Category 2 issues defined in
25 Section 51.53(c)(3)(ii) presented in the McGuire ER (Duke 2001a). As the NRC review
26 guidance recognizes, if no significant offsite impacts occur in connection with the proposed
27 action, then no member of the public will be substantially affected. Therefore, there can be no
28 disproportionately high and/or adverse impacts or effects on any member of the public, including
29 minority and low-income populations, resulting from the renewal of the McGuire licenses.

30
31 The staff found no unusual resource dependencies or practices, such as subsistence agricul-
32 ture, hunting, or fishing, through which minority or low-income populations could be dispropor-
33 tionately adversely impacted. In addition, the staff did not identify any location-dependent
34 disproportionately adverse impacts affecting these minority and low-income populations. The

(a) Figure 4-2 was prepared using 1990 income data because the 2000 census income data are not yet available.

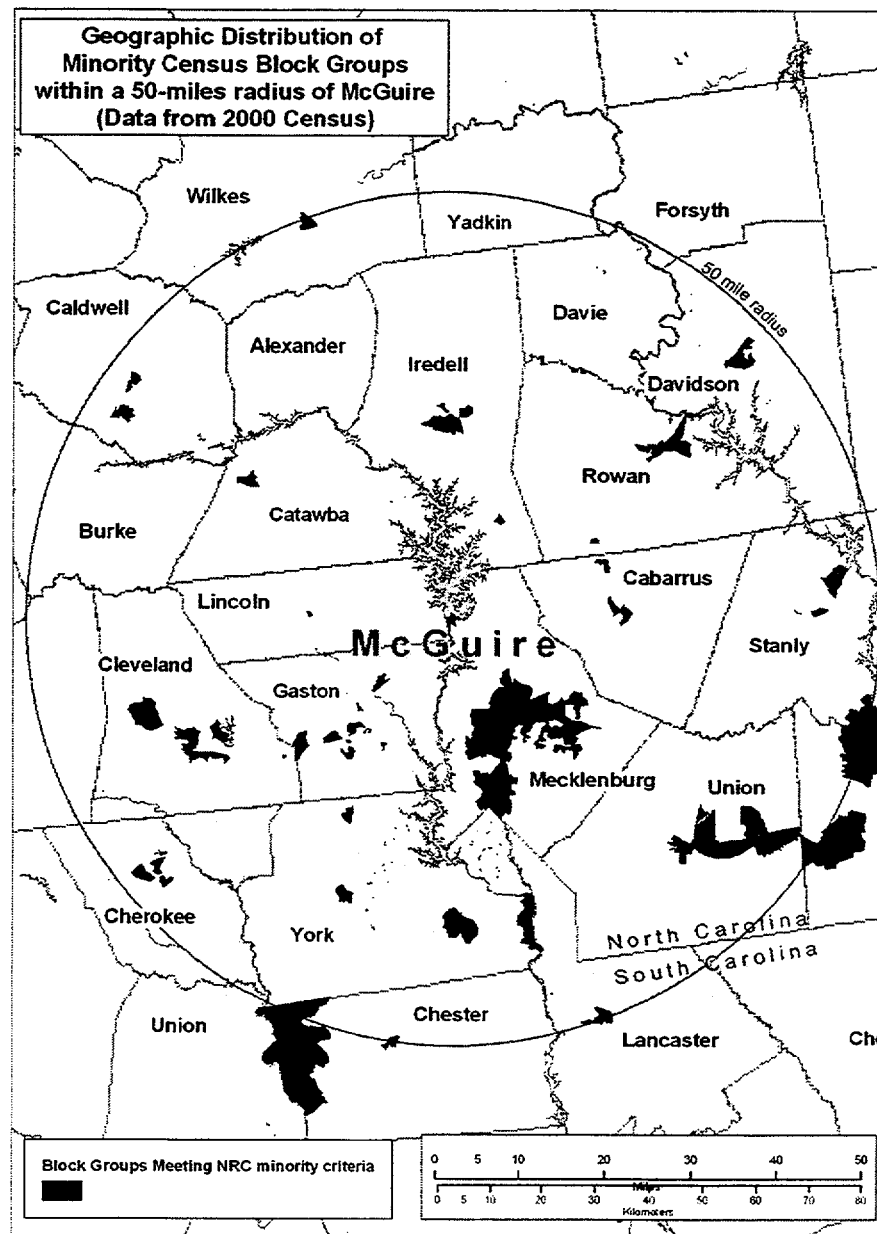
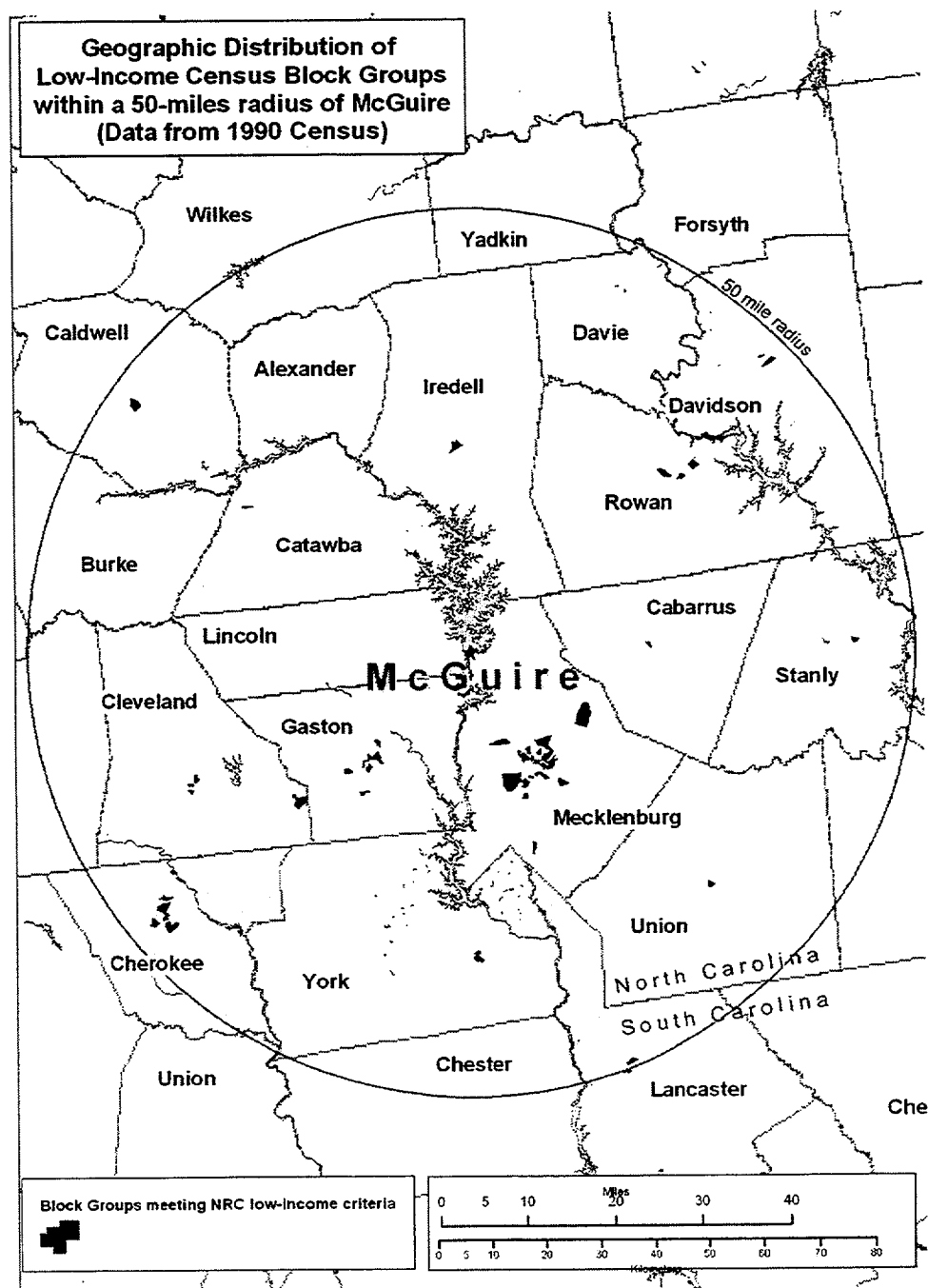


Figure 4-1.
Census 2000 Block Groups Identified as Meeting NRC Criteria for Minority Status in an 80-km (50-mi) Area Around McGuire



1 **Figure 4-2. Census 2000 Block Groups Identified as Meeting NRC Criteria for Low-Income**
 2 **Status in an 80-km (50-mi) Area Around McGuire**

staff concludes that offsite impacts from McGuire to minority and low-income populations would be SMALL, and no additional mitigation actions are warranted.

4.5 Groundwater Use and Quality

Category 1 issues in 10 CFR Part 51, Subpart A, Appendix B, Table B-1 that are applicable to McGuire groundwater use and quality are listed in Table 4-9. Duke stated in its ER that “no new information existed for the issues that would invalidate the GEIS conclusions” (Duke 2001a). The staff has not identified any significant new information during its independent review of the McGuire ER (Duke 2001a), the staff’s site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts related to this issue beyond those discussed in the GEIS. For this issue, the GEIS concluded that the impacts are SMALL, and plant-specific mitigation measures are not likely to be sufficiently beneficial to be warranted.

Table 4-9. Category 1 Issue Applicable to Groundwater Use and Quality During the Renewal Term

ISSUE—10 CFR Part 51, Subpart A, Appendix B, Table B-1	GEIS Section
GROUNDWATER USE AND QUALITY	
Groundwater-use conflicts (potable and service water; plants that use [$<$]100 gpm).	4.8.1.1

A brief description of the staff’s review and the GEIS conclusions, as codified in Table B-1, for each of these issues follows.

- Groundwater-use conflicts (potable and service water; plants that use $<$ 100 gpm).
Based on information in the GEIS, the Commission found that

Plants using less than 100 gpm are not expected to cause any ground-water use conflicts.

As discussed in Section 2.2.2, McGuire groundwater use is less than 0.068 m³/s (100 gpm). The staff has not identified any significant new information during its independent review of the McGuire ER, the staff’s site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no groundwater-use conflicts during the renewal term beyond those discussed in the GEIS.

There are no Category 2 issues related to groundwater use and quality for McGuire.

4.6 Threatened or Endangered Species

Threatened or endangered species is listed as a Category 2 issue in 10 CFR Part 51, Subpart A, Appendix B, Table B-1. This issue is listed in Table 4-10.

Table 4-10. Category 2 Issue Applicable to Threatened or Endangered Species During the Renewal Term for McGuire

ISSUE—10 CFR Part 51, Subpart A, Appendix B, Table B-1	GEIS Section	10 CFR 51.53(c)(3)(ii) Subparagraph	SEIS Section
THREATENED OR ENDANGERED SPECIES (FOR ALL PLANTS)			
Threatened or endangered species	4.1	E	4.6

This issue requires consultation with appropriate agencies to determine whether threatened or endangered species are present and whether they would be adversely affected by continued operation of the nuclear plant during the license renewal term. NRC Staff initiated informal consultation with the FWS by letter requesting information on species protected under the Endangered Species Act that occur in the vicinity of the McGuire site. The FWS responded by letter (Cole 2001) indicating no known occurrences on the McGuire site. The presence of threatened or endangered species in the vicinity of the McGuire site is discussed in Sections 2.2.5 and 2.2.6.

4.6.1 Aquatic Species

As described in Section 2.2.5, the only Federally or State-listed threatened or endangered aquatic species with potential to inhabit waters near McGuire, the Carolina heelsplitter (*Lasmigona decorata*), is not present in the vicinity of the plant (Fridell 2001) and does not occur in impounded water. Thus, continued operation of the plant should not result in impacts to threatened or endangered aquatic species.

Based on these considerations, the staff has determined that the continued operation of McGuire and the continued maintenance of the transmission lines will not impact listed aquatic species.

4.6.2 Terrestrial Species

A field survey for species of concern was conducted within the McGuire exclusion area and on the related transmission line rights-of-way in summer and fall 2000. During this survey, no Federally listed threatened or endangered species were located (Gaddy 2001). In a letter dated November 1, 2001, the FWS (Cole 2001) concurred with the findings of the survey report (Gaddy 2001).

However, the bald eagle is known to visit infrequently the shore of Lake Norman. Based on a preliminary review of the applicant's report and the staff's independent analysis, the NRC staff concluded that continued operation of the McGuire site and related transmission corridors under license renewal will not adversely impact the bald eagle.

Schweinitz's sunflower (*Helianthus schweinitzii*) (Federal endangered) occurs in relatively open habitats, such as road and power line rights-of-way, early successional fields, forest ecotonal margins, and forest clearings. Georgia aster (*Aster georgianus*) (Federal threatened) occurs in dry open woods along roadsides, woodland borders, old fields, and pastures (Cole 2001). Neither of these species is currently known to occur on the McGuire site nor is expected to colonize this area due to lack of appropriate soils (Gaddy 2001).

Based on a preliminary review of the applicant's report and the staff's independent analysis, the NRC staff concluded that continued operation of the McGuire site and related transmission corridors under license renewal will not adversely impact Schweinitz's sunflower and Georgia aster.

It is the staff's determination that the impact to threatened or endangered species of an additional 20 years of maintenance activities for the transmission lines would be SMALL, and additional mitigation is not required.

4.7 Evaluation of Potential New and Significant Information on Impacts of Operations During the Renewal Term

The staff has not identified new and significant information on environmental issues listed in 10 CFR Part 51, Subpart A, Appendix B, Table B-1, related to operation during the renewal term. The staff reviewed the discussion of environmental impacts associated with operation during the renewal term in the GEIS and conducted its own independent review, including the public scoping meetings, to identify issues with significant new information. Processes for

identification and evaluation of new information are described in Chapter 1.0 under License Renewal Evaluation Process.

4.8 Summary of Impacts of Operations During the Renewal Term

Neither Duke nor the staff is aware of information that is both new and significant related to any of the applicable Category 1 issues associated with the McGuire operation during the renewal term. Consequently, the staff concludes that the environmental impacts associated with these issues are bounded by the impacts described in the GEIS. For each of these issues, the GEIS concluded that the impacts would be SMALL and that "plant-specific mitigation measures are not likely to be sufficiently beneficial to warrant implementation."

Plant-specific environmental evaluations were conducted for 11 Category 2 issues applicable to McGuire operation during the renewal term and for environmental justice. For all 11 issues and environmental justice, the staff concluded that the potential environmental impact of renewal term operations of McGuire would be of SMALL significance in the context of the standards set forth in the GEIS and that mitigation would not be warranted. In addition, the staff determined that a consensus has not been reached by appropriate Federal health agencies that there are adverse effects from electromagnetic fields. Therefore, the staff did not conduct an evaluation of this issue.

4.9 References

10 CFR 51. Code of Federal Regulations Title 10, *Energy*, Part 51, "Environmental Protection Regulations for Domestic Licensing and Related Regulatory Functions."

36 CFR 800. Code of Federal Regulations, Title 36, *Parks, Forests, and Public Property*, Part 800, "Advisory Council on Historic Preservation."

Brook, D. 2000. Letter from Deputy State Historic Preservation Officer (signed by Renee Gledhill-Early), North Carolina Department of Cultural Resources, to J. R. Huff, Duke Power. (January 31, 2000).

Cole, B. 2001. Letter from U.S. Fish and Wildlife Service State Supervisor, Asheville Field Office, Asheville, North Carolina to Cynthia A. Carpenter, Chief, Risk Informed Initiatives, Environmental Decommissioning and Rulemaking Branch, Division of Nuclear Regulatory

1 Improvement Programs, Office of Nuclear Reactor Regulation, Nuclear Regulatory Commission.
2 Subject: "McGuire Nuclear Station, Units 1 and 2, License Renewal Project, Mecklenburg
3 County, North Carolina (Docket Nos. 50-369 and 50-370)." November 1, 2001.

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5.0 Environmental Impacts of Postulated Accidents

Environmental issues associated with postulated accidents were discussed in the *Generic Environmental Impact Statement for License Renewal of Nuclear Plants* (GEIS), NUREG-1437, Volumes 1 and 2 (NRC 1996, 1999a).^(a) The GEIS includes a determination of whether the analysis of the environmental issues could be applied to all plants and whether additional mitigation measures would be warranted. Issues are then assigned a Category 1 or a Category 2 designation. As set forth in the GEIS, Category 1 issues are those that meet all of the following criteria:

- (1) The environmental impacts associated with the issue have been determined to apply either to all plants or, for some issues, to plants having a specific type of cooling system or other specified plant or site characteristic.
- (2) A single significance level (i.e., SMALL, MODERATE, or LARGE) has been assigned to the impacts (except for collective offsite radiological impacts from the fuel cycle and from high-level waste and spent fuel disposal).
- (3) Mitigation of adverse impacts associated with the issue has been considered in the analysis, and it has been determined that additional plant-specific mitigation measures are likely not to be sufficiently beneficial to warrant implementation.

For issues that meet the three Category 1 criteria, no additional plant-specific analysis is required unless new and significant information is identified.

Category 2 issues are those that do not meet one or more of the criteria for Category 1, and therefore, additional plant-specific review of these issues is required.

This chapter describes the environmental impacts from postulated accidents that might occur during the license renewal term.

5.1 Postulated Plant Accidents

Two classes of accidents are evaluated in the GEIS. These are design-basis accidents (DBAs) and severe accidents, as discussed in the following sections.

(a) The GEIS was originally issued in 1996. Addendum 1 to the GEIS was issued in 1999. Hereafter, all references to the "GEIS" include the GEIS and its Addendum 1.

Environmental Impacts of Postulated Accidents

Design-Basis Accidents

To receive NRC approval to operate a nuclear power facility, an applicant for an initial operating license must submit a Safety Analysis Report (SAR) as part of its application. The SAR presents the design criteria and design information for the proposed reactor and comprehensive data on the proposed site. The SAR also discusses various hypothetical accident situations and the safety features that are provided to prevent and mitigate accidents. The staff reviews the application to determine whether the plant design meets the Commission's regulations and requirements and includes, in part, the nuclear plant design and its anticipated response to an accident.

DBAs are those that both the licensee and the staff evaluate to ensure that the plant can withstand normal accidents and abnormal transients and a broad spectrum of postulated accidents without undue hazard to the health and safety of the public. A number of these postulated accidents are not expected to occur during the life of the plant but are evaluated to establish the design basis for the preventive and mitigative safety systems of the facility. The acceptance criteria for DBAs are described in 10 CFR Part 50 and 10 CFR Part 100.

The environmental impacts of DBAs are evaluated during the initial licensing process, and the ability of the plant to withstand these accidents is demonstrated to be acceptable before issuance of the operating license (OL). The results of these evaluations are found in this section and in license documentation such as the applicant's Final Safety Analysis Report (FSAR), the staff's Safety Evaluation Report (SER), the Final Environmental Statement (FES), and Section 5.1 of this Supplemental Environmental Impact Statement (SEIS). A licensee is required to maintain the acceptable design and performance criteria throughout the life of the plant including any extended-life operation. The consequences for these events are evaluated for the hypothetical maximum exposed individual; as such, changes in the plant environment will not affect these evaluations. Because of the requirements that continuous acceptability of the consequences and aging management programs be in effect for license renewal, the environmental impacts as calculated for DBAs should not differ significantly from initial licensing assessments over the life of the plant, including the license renewal period. Accordingly, the design of the plant relative to DBAs during the extended period is considered to remain acceptable and the environmental impacts of those accidents were not examined further in the GEIS.

The Commission has determined that the environmental impacts of DBAs are of SMALL significance for all plants because the plants were designed to successfully withstand these accidents. Therefore, for the purposes of license renewal, design-basis events are designated as a Category 1 issue in 10 CFR Part 51, Subpart A, Appendix B, Table B-1. This issue, applicable to McGuire Nuclear Station, Units 1 and 2 (McGuire), is listed in Table 5-1. The early

resolution of the DBAs makes them a part of the current licensing basis of the plant; the current licensing basis of the plant is to be maintained by the licensee under its current license and, therefore, under the provisions of 10 CFR 54.30, is not subject to review under license renewal.

Table 5-1. Category 1 Issue Applicable to Postulated Accidents During the Renewal Term

ISSUE—10 CFR Part 51, Subpart A, Appendix B, Table B-1	GEIS Sections
POSTULATED ACCIDENTS	
Design-basis accidents (DBAs)	5.3.2; 5.5.1

Based on information in the GEIS, the Commission found that

The NRC staff has concluded that the environmental impacts of design-basis accidents are of small significance for all plants.

In its Environmental Report (ER), Duke Energy Corporation (Duke) stated that “no new information existed for the issues that would invalidate the GEIS conclusions (Duke 2001).” The staff has not identified any significant new information during its independent review of the McGuire ER (Duke 2001), the staff’s site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts related to this issue beyond those discussed in the GEIS.

Severe Accidents

Severe nuclear accidents are those that are more severe than DBAs because they could result in substantial damage to the reactor core, whether or not there are serious offsite consequences. In the GEIS, the staff assessed the impacts of severe accidents during the license renewal period, using the results of existing analyses and site-specific information to conservatively predict the environmental impacts of severe accidents for each plant during the renewal period.

Severe accidents initiated by external phenomena such as tornadoes, floods, earthquakes, and fires have not traditionally been discussed in quantitative terms in FESs and were not considered specifically for the McGuire site in the GEIS (NRC 1996). However, in the GEIS, the staff did evaluate existing impact assessments performed by the NRC and by the industry at 44 nuclear plants in the United States and concluded that the risk from beyond-design-basis earthquakes at existing nuclear power plants is SMALL. Additionally, the staff concluded that

Environmental Impacts of Postulated Accidents

the risks from other external events are adequately addressed by a generic consideration of internally initiated severe accidents.

Based on information in the GEIS, the Commission found that

The probability-weighted consequences of atmospheric releases, fallout onto open bodies of water, releases to groundwater, and societal and economic impacts from severe accidents are small for all plants. However, alternatives to mitigate severe accidents must be considered for all plants that have not considered such alternatives.

Therefore, the Commission has designated mitigation of severe accidents as a Category 2 issue in 10 CFR Part 51, Subpart A, Appendix B, Table B-1. This issue, applicable to McGuire, is listed in Table 5-2.

Table 5-2. Category 2 Issue Applicable to Postulated Accidents During the Renewal Term

ISSUE—10 CFR Part 51, Subpart A, Appendix B, Table B-1	GEIS Sections	10 CFR 51.53(c)(3)(ii) Subparagraph	SEIS Section
POSTULATED ACCIDENTS			
Severe Accidents	5.3.3; 5.3.3.2; 5.3.3.3; 5.3.3.4; 5.3.3.5; 5.4; 5.5.2	L	5.2

The staff has not identified any significant new information with regard to the consequences from severe accidents during its independent review of the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of severe accidents beyond those discussed in the GEIS. However, in accordance with 10 CFR 51.53(c)(ii)(L), the staff has reviewed severe accident mitigation alternatives (SAMAs) for McGuire. The results of its review are discussed in Section 5.2.

5.2 Severe Accident Mitigation Alternatives (SAMAs)

10 CFR 51.53(c)(3)(ii)(L) requires that license renewal applicants consider alternatives to mitigate severe accidents if the staff has not previously evaluated SAMAs for the applicant's plant in an EIS or related supplement or in an environmental assessment. The purpose of this

consideration is to ensure that plant changes (i.e., hardware, procedures, and training) with the potential for improving severe accident safety performance are identified and evaluated. SAMAs have not been previously considered for McGuire; therefore, the remainder of Chapter 5 addresses those alternatives.

5.2.1 Introduction

Duke submitted an assessment of SAMAs for McGuire as part of the ER (Duke 2001). The assessment was based on Revision 2 of the McGuire Probabilistic Risk Assessment (McGuire PRA, Revision 2) (Duke 1998), which is a full scope Level 3 PRA analysis with the analysis of both internal and external events. The internal events analysis is an updated version of the Individual Plant Examination (IPE) model (Duke Power 1991), and the external events analysis is based on the Individual Plant Examination for External Events (IPEEE) model (Duke Power 1994). In identifying and evaluating potential SAMAs, Duke took into consideration the insights from the McGuire PRA, as well as other studies, such as the Watts Bar Severe Accident Mitigation Design Alternatives (SAMDA) Analysis (NRC 1995a) and NUREG-1560 (NRC 1997c). Duke concluded that none of the candidate SAMAs evaluated were cost effective for McGuire.

Based on a review of the initial SAMA assessment, the staff issued a request for additional information (RAI) to Duke by letter dated November 19, 2001 (NRC 2001). Key questions concerned further information on several candidate SAMAs, especially those that mitigate the consequences of a station blackout (SBO) event; details on the updated PRA used for the SAMA analysis, including results as they pertain to containment failure and releases; and the impact of including elements of averted risk that were omitted in the ER. By a letter dated January 31, 2002, Duke submitted additional information (Duke 2002), which provided details on the updated PRA, the requested PRA results, and other information identified in the RAI (NRC 2001). Duke provided additional clarification in a conference call on February 25, 2002 (NRC 2002a). In these responses, Duke included supplemental tables showing the impacts of including averted replacement power costs for SAMAs that have the potential to reduce core damage frequencies and averted offsite property damage costs for SAMAs that have the potential to improve containment performance—both of which were omitted in the original analysis. Also, Duke presented its position on the value of providing back-up hydrogen control capability during SBO events. Duke's responses addressed the staff's concerns and reaffirmed that none of the SAMAs would be cost-beneficial. However, based on review of the cost and benefit information provided by Duke, the staff concludes that one SAMA appears to be cost-beneficial. This SAMA, which involves plant and procedure modifications to enable the existing hydrogen control (igniter) system to be powered from an ac-independent power source in SBO events, has not been implemented at McGuire. This issue is currently being addressed by the NRC as part of the resolution of Generic Safety Issue 189 - Susceptibility of Ice Condenser and

Mark III Containments to Early Failure from Hydrogen Combustion During a Severe Accident (NRC 2002b).

The Staff's assessment of SAMAs for McGuire is presented below.

5.2.2 Estimate of Risk for McGuire Units 1 and 2

Duke's estimates of offsite risk at McGuire are summarized below. The summary is followed by the Staff's review of Duke's risk estimates.

5.2.2.1 Duke's Risk Estimates

The McGuire PRA model, which forms the basis for the SAMA analysis, is a Level 3 risk analysis; i.e., it includes the treatment of core damage frequency, containment performance, and offsite consequences. The model, which Duke refers to as PRA, Revision 2 (Duke 1998), consists of an internal events analysis based on an updated version of the original IPE (McGuire PRA, Revision 1) (Duke Power 1991) and an external events analysis based on the current version of the IPEEE (Duke Power 1994). The calculated total core damage frequency (CDF) for internal and external events in Revision 2 of the McGuire PRA is $4.9\text{E-}5$ per year.

Since the McGuire PRA is a "living" PRA, the original version of the IPE has been updated to reflect various design and procedural changes, such as those related to the improvements identified in the IPE, comments received during the McGuire peer review process, and operational experience since 1991. The CDF for internal and external events was reduced from $7.4\text{E-}05$ per year (Revision 1) to $4.9\text{E-}5$ per year (Revision 2). The Level 1 PRA changes associated with the McGuire PRA Revision 2 model included

- incorporation of updated data for component reliability, unavailabilities, initiating event frequencies, common cause failures, and human error probabilities
- conversion from a sequence based solution to a single top fault tree
- modifications to reflect changes to the plant configuration.

The most significant data changes are those related to diesel generator (DG) performance. Following the IPE, Duke proceeded with a program to improve the DG reliability at McGuire. The reliability improvement that occurred significantly reduced the CDF contributed by the loss of offsite power (LOOP) and tornado initiators. To a lesser extent, the seismic results are also impacted by the DG reliability data. The net effect is that the total CDF for SBO sequences (internal and external events) was reduced from approximately $4.1\text{E-}5$ per year in the IPE and

IPEEE to $2.3\text{E-}5$ per year in PRA Revision 2. Another important change occurred in the interfacing system loss-of-coolant accident (ISLOCA) evaluation. The generic database adopted for the Revision 2 analysis had significantly higher failure rates for valve ruptures. This resulted in a significant increase in the CDF contributed by the ISLOCA, an important risk contributor.

The breakdown of the CDF from Revision 2 to the PRA is provided in Table 5-3. Internal event initiators represent about 57 percent of the total CDF and are composed of transients (31 percent of total CDF), loss-of-coolant accidents (LOCAs) (22 percent of total CDF), and reactor pressure vessel rupture (2 percent of total CDF). Remaining contributors together account for less than 3 percent of total CDF. External event initiators represent about 43 percent of the total CDF and are composed of seismic initiators (22 percent of total CDF), tornado initiators (13 percent of total CDF), and fire initiators (6 percent of the total CDF). Although not explicitly reported in Table 5-3, SBO events account for 47 percent of the total CDF for internal and external events in Revision 2 of the PRA.

Table 5-3. McGuire Core Damage Frequency (Revision 2 of PRA)

Initiating Event	Frequency (per reactor-year)	% of Total CDF
Transients	$1.5\text{E-}05$	31
Loss-of-coolant accident (LOCA)	$1.1\text{E-}05$	22
Internal flood	$8.7\text{E-}07$	2
Anticipated transient without scram	$1.5\text{E-}07$	<1
Steam generator tube rupture (SGTR)	$7.8\text{E-}10$	<1
Reactor pressure vessel rupture	$1.0\text{E-}06$	2
Interfacing system LOCA	$2.2\text{E-}07$	<1
CDF from internal events	$2.8\text{E-}05$	57
Seismic	$1.1\text{E-}05$	22
Tornado	$6.5\text{E-}06$	13
Fire	$2.9\text{E-}06$	6
CDF from external events	$2.1\text{E-}05$	43
Total CDF	$4.9\text{E-}05$	100

The Level 2 (also called containment performance) portion of the McGuire PRA model, Revision 2, is essentially the same as the IPE Level 2 analysis. However, the following changes were made:

- modifications to reflect an emergency operating procedure change that reduced the likelihood of restarting a reactor coolant pump following core damage, thus reducing the potential for thermally induced steam generator tube rupture

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- modification of the containment event tree (CET) logic regarding the potential for corium contact with the containment liner
- modification of the CET logic and quantification to reflect that the refueling water storage tank inventory would drain through a failed reactor vessel in some sequences (e.g., SBO).

These changes resulted in a large decrease in the potential for thermally-induced steam generator tube ruptures, a slight increase in the potential for early containment failure as a result of corium contact with the containment liner and a reduction in basemat melt-through due to reactor cavity flooding via the reactor vessel breach.

The offsite consequences and economic impact analyses (i.e., Level 3 PRA Analyses) were carried out using the NRC-developed MELCOR Accident Consequence Code System 2 (MACCS2) code. Inputs for this analysis include plant and site specific input values for core radionuclide inventory, source term and release fractions, meteorological data, projected population distribution, and emergency response evacuation modeling.

Duke estimated the dose to the population within 80 km (50 mi) of the McGuire site from all initiators (internal and external) to be about 0.135 person-sieverts (Sv) (13.5 person-rem) per year (Duke 2001). The breakdown of the total population dose by containment end-state is summarized in Table 5-4. Internal events account for approximately 0.006 person-Sv (6.0 person-rem) per year, and external events account for approximately 0.0075 person-Sv

Table 5-4. Breakdown of Population Dose by Containment End-State
[Total dose = 0.135 person-Sv (13.5 person-rem) per year]

Containment End State	% of Total Dose Internal Initiators	% of Total Dose External Initiators	% of Total Dose All Initiators
Steam generator tube rupture*	<0.1	<0.1	<0.1
Interfacing system LOCA ^(a)	19.4	0.0	19.4
Containment isolation failure	0.1	0.3	0.4
Early containment failure	8.5	32.1	40.6
Late containment failure	15.9	23.3	39.2
Basemat melt-through	<0.1	<0.1	<0.1
No containment failure	0.3	0.1	0.4
Total	44.2	55.8	100

(a) Containment bypass events

(7.5 person-rem) per year. As can be seen from this table, early and late containment failures account for the majority of the population dose.

5.2.2.2 Review of Duke's Risk Estimates

Duke's estimate of offsite risk at McGuire is based on the Revision 2 of the McGuire PRA and a separate MACCS2 analysis. For the purposes of this review, the Staff considered the following major elements:

- the Level 1 and 2 risk models that form the bases for the November 1991 IPE submittal (Duke 1991)
- the major modifications to the IPE models that have been incorporated in Revision 2 of the PRA (Duke 1998)
- the external events models that form the basis for the June 1994 IPEEE submittal (Duke 1994)
- the analyses performed to translate fission product release frequencies from the Level 2 PRA model into offsite consequence measures (Duke 2001).

The Staff reviewed each of these analyses to determine the acceptability of Duke's risk estimates for the SAMA analysis, as summarized below.

The Staff's review of the McGuire IPE is described in a Staff report dated June 30, 1994 (NRC 1994). In that review, the Staff evaluated the methodology, models, data, and assumptions used to estimate the CDF and characterize containment performance and fission product releases. The Staff concluded that Duke's analysis met the intent of Generic Letter 88-20 (NRC 1988), which means the IPE was of adequate quality to be used to look for design or operational vulnerabilities. The Staff's review primarily focused on the licensee's ability to examine McGuire for severe accident vulnerabilities and not specifically on the detailed findings or quantification estimates. Overall, the Staff concluded that the McGuire IPE was of adequate quality to be used as a tool in searching for areas with high potential for risk reduction and to assess such risk reductions, especially when the risk models are used in conjunction with insights, such as those from risk importance, sensitivity, and uncertainty analyses.

The staff's review of the McGuire IPEEE is described in an evaluation report dated February 16, 1999 (NRC 1999b). Duke did not identify any fundamental weaknesses or vulnerabilities to severe accident risk with regard to the external events. In the safety evaluation report, the Staff concluded that the IPEEE met the intent of Supplement 4 to Generic Letter 88-20 (NRC 1991) and that the licensee's IPEEE process is capable of identifying the most likely severe accidents and severe accident vulnerabilities.

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1 In a RAI (NRC 2001), the staff questioned why the CDF for steam generator tube
2 rupture events in Revision 2 to the PRA is so low relative to other pressurized-water reactor
3 (PWR) PRAs. In response (Duke 2002a), Duke stated that

4
5 The McGuire SGTR model incorporated in both the IPE and in the 1997 update relied
6 upon success criteria established during the IPE development. Where applicable, the
7 system success criteria were established with the then current version of the MAAP
8 [Modular Accident Analysis Program] code. Furthermore, a sequence was categorized
9 as a success because core damage occurred beyond 24 hours, even though a safe
10 stable state had not been attained, this is inconsistent with what is now the generally
11 accepted industry practice. As a result of comments received during the McGuire peer
12 review process, these success criteria were revisited. The new MAAP results showed
13 core damage to occur where the original analysis did not. The outdated success criteria
14 are judged to be the most significant contributors to the comparatively low SGTR
15 initiated CDF previously reported. The SGTR analysis is being completely revisited in
16 Revision 3 to the McGuire PRA, which is still in development. This new analysis
17 estimates the CDF for SGTR at $5.3\text{E-}07$ per year, which is more in line with similar
18 plants.

19
20 In a February 7, 2002, telephone conference with Duke, the staff questioned the impact that
21 other Revision 3 PRA results might have on the conclusions drawn in the McGuire ER, because
22 the change for the SGTR event was not trivial. In response (NRC 2002a), Duke provided CDF
23 estimates from Revision 3 of the McGuire Level 1 PRA, broken out by the major contributors.
24 Peer review of the Level 2 and 3 portions of the PRA Revision 3 had not yet been completed.
25 Thus, revised Level 2 and 3 information was not provided. A comparison of the CDF results
26 from the various versions of the McGuire PRA is provided in Table 5-5.

27
28 Based on a comparison of the frequency of major contributors to CDF, the following key
29 differences were noted:

- 30
- 31 • The SGTR frequency in Revision 3 is more than a factor of 600 larger than in Revision 2
32 ($5.3\text{E-}7$ per year versus $7.8\text{E-}10$ per year). This increase is due to the use of revised,
33 more technically-supported success criteria as discussed above.
 - 34
 - 35 • The SBO frequency in Revision 3 is more than a factor of two smaller than in Revision 2
36 ($1.0\text{E-}5$ per year versus $2.3\text{E-}5$ per year). This reduction is due to credit taken for
37 installing improved reactor coolant pump O-ring seals that would be capable of
38 withstanding higher temperatures and would have a higher likelihood of remaining intact
39 under loss of seal-cooling conditions.

Table 5-5. Comparison of CDF Results by Accident Initiator or Sequence

Accident Initiator/Sequence	PRA, Rev. 1 (IPE)	PRA, Rev. 2	PRA, Rev. 3
SBO	4.1E-5 (internal & external events) 9.5E-6 (internal only)	2.3E-5 (internal & external events)	1.0E-5 (internal & external events)
LOOP	1.1E-5	2.6E-6	1.3E-6
Internal Floods	--	8.7E-7	5.4E-6
Transients	--	1.5E-5	2.8E-6
LOCAs	--	1.1E-5	1.9E-5
RPV ^(a)	--	1.0E-6	1.0E-6
SGTR	--	7.8E-10	5.3E-7
ATWS ^(b)	--	1.5E-7	5.3E-7
ISLOCA	--	2.2E-7	9.8E-7
CDF from internal events	4.0E-5	2.8E-5	3.0E-5
Seismic	1.1E-5	1.1E-5	8.9E-6
Tornado	1.9E-5	6.5E-6	1.5E-6
Fire	2.3E-7	2.9E-6	6.3E-6
Total CDF	7.0E-5		4.6E-5
(a) reactor pressure vessel			
(b) anticipated transients without scram			

The impact of the revised SGTR and SBO frequencies on the risk reduction estimates for related SAMAs was considered in the staff's review (see Sections 5.2.4 and 5.2.6.2). The frequency of other CDF contributors was impacted to a much lesser degree, and these changes are not expected to alter results of the SAMA analysis.

The staff reviewed the process used by Duke to extend the containment performance (Level 2) portion of the IPE to the offsite consequence (Level 3) assessment. This included consideration of the source terms used to characterize fission product releases for each containment release category and the major input assumptions used in the offsite consequence analyses. This information is provided in Section 6 of Duke's IPE submittal. Duke used the MAAP code to analyze postulated accidents and develop radiological source terms for each of 31 containment release categories used to represent the containment end-states. These

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1 source terms were incorporated as input to the MACCS2 analysis. The MACCS2 code is the
2 current standard for assessing consequences of accidents at nuclear power plants. The Staff
3 reviewed Duke's source term estimates for the major release categories and found these
4 predictions to be in reasonable agreement with estimates from NUREG-1150 (NRC 1990a) for
5 the closest corresponding release scenarios. The Staff concludes that the assignment of
6 source terms is acceptable.

7
8 The plant-specific input to the MACCS2 code includes the McGuire reactor core radionuclide
9 inventory, emergency response evacuation modeling based on McGuire evacuation time
10 estimate studies, release category source terms from the McGuire PRA Revision 2 analysis
11 (same as the source terms used in the IPE), site-specific meteorological data, and projected
12 population distribution within a 80 km (50 mile) radius for the year 2040.

13
14 MACCS2 requires a file of hourly meteorological data consisting of wind speed, wind direction,
15 atmospheric stability category, and precipitation. For the McGuire SAMA analysis, meteorological
16 data was obtained from the meteorological tower located on the McGuire site; the
17 meteorological data used in MACCS2 contained data for one year, January 1 through
18 December 31, 1999.

19
20 The McGuire PRA Revision 2 and the SAMA offsite consequence analyses use three distinct
21 evacuation schemes in order to adequately represent evacuation time estimates for the
22 permanent resident population, the transient population, and the special facility population (e.g.,
23 schools, hospitals, etc.). The three groups are defined by the time delay from initial notification
24 to start of evacuation. For each evacuation scheme, the fraction of the population starting their
25 evacuation is included. For the permanent resident evacuation schemes, it was assumed that 5
26 percent of the population would delay evacuation for 24 hours after being warned to evacuate.
27 The delay time and fraction of population for the remaining two schemes were developed from
28 information given in the latest update to the McGuire evacuation time estimate study for the
29 16 km (10-mi) Emergency Planning Zone (EPZ). The evacuation schemes include additional
30 information such as evacuation distance, average evacuation speed, sheltering, and shielding
31 considerations. In the McGuire evacuation model, only the 10-mile EPZ is assumed to be
32 involved in the initial evacuation. The MACCS2 model assumes that persons outside of the
33 10-mile EPZ will wait 24 hours before evacuating (provided that radiological conditions warrant
34 evacuation).

35
36 The staff reviewed the Duke responses (Duke 2002) to questions regarding meteorological
37 data, population data and emergency planning. Those responses confirmed that Duke used
38 appropriate values for the consequence analysis.

39
40 The staff concludes that the methodology used by Duke to estimate the CDF and offsite
41 consequences for McGuire provides an acceptable basis from which to proceed with an

assessment of the risk reduction potential for candidate SAMAs. Additionally, the risk profile used is similar to other PWRs with ice condenser containments. The staff based its assessment of offsite risk on the CDF and offsite doses reported by Duke, but also considered the impact that the use of CDF estimates from Revision 3 of the PRA might have on the risk results.

5.2.3 Potential Design Improvements

This section discusses the process for identifying potential design improvements, the staff's evaluation of this process, and the design improvements evaluated in detail by Duke.

5.2.3.1 Process for Identifying Potential Design Improvements

Duke's process for identifying potential plant improvements consisted of the following elements:

- The core damage cut sets from Revision 2 of the McGuire PRA were reviewed to identify potential SAMAs that could reduce CDF.
- The Fussell-Vesely (F-V) importance measures were evaluated for the basic events (including initiating events, random failure events, human error events, and maintenance and testing unavailabilities), and the importance ranking was examined to identify any events of significant F-V importance.
- Potential enhancements to reduce containment failure modes of concern for McGuire (including early containment failure, containment isolation failure, and containment bypass) were reviewed for possible implementation.

In addition, Duke reviewed the Watts Bar SAMDA analysis (NRC 1995a) and insights from the staff's report on the IPE (NRC 1997c) to identify additional SAMAs.

As a starting point for the core damage cut set review, Duke developed a listing of the top 100 cut sets (severe accident sequences) based on internal initiators and the top 100 cut sets for external initiators. These 200 sequences include all potential core damage sequences with at least a 0.06 percent contribution to the total CDF. Additionally, some cut sets contributing as little as 0.05 percent to the total CDF were considered. Duke reviewed the cut sets to identify potential SAMAs that could reduce CDF. A cutoff value of $3.5\text{E-}7$ per year (for internal and external event initiators) was used to screen events. To account for the cumulative effect of cut sets below this cutoff value, the basic events importance measure was also used to identify potential enhancements, as discussed below. Duke indicated in responses to the RAIs (Duke 2002) that the estimated CDF for the 200 cut sets is $4.4\text{E-}5$ per year, which is about 90 percent of the total CDF.

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1 For each seismic initiator cut set, Duke calculated the associated offsite risk based on the
2 population dose and CDF for the plant damage states (PDSs) attributable to the seismic
3 initiator. Duke conservatively assumed that the implementation of plant enhancements for
4 seismic events would completely eliminate the seismic risk and calculated the present worth of
5 the averted risk based on a \$200,000 per person-Sv (\$2000 per person-rem) conversion factor,
6 a discount factor of 7 percent, and a 20-year license renewal period. This process was
7 repeated for each of the remaining seismic initiator cutsets above the cutoff frequency. The
8 present worth of averted risk for all of the seismic cutsets combined was estimated to be about
9 \$275,000 (not including the cost of replacement power and offsite property damage, the
10 significance of which is discussed in Section 5.2.6.2). On the basis of the small risk reduction
11 achievable [0.041 person-Sv (4.1 person-rem)] and the large costs associated with substantial
12 seismic upgrades (estimated at several million dollars), Duke eliminated seismic SAMAs from
13 further consideration.

14
15 Duke reviewed the F-V Basic Event Importance Ranking presented in the McGuire PRA report,
16 Revision 2, and identified several basic events for further consideration. These included
17 internal event initiators, seismic-related events, equipment failures, and human-error events.
18 Seismic-related events were not evaluated further for the reasons discussed above. Seven
19 potential enhancements to reduce CDF were identified through this process and are presented
20 in Table 5-6.

21
22 In the ER, Duke identified the installation of back-up power to the igniters and the installation of
23 back-up power to air return fans as two separate SAMAs. However, in responses to staff
24 RAIs, Duke indicated that the availability of air return fans would be essential to the effective
25 operation of igniters in an SBO; therefore, Duke treated the combined modification as a single
26 SAMA. Accordingly, these two hydrogen control related SAMAs are shown as a single SAMA in
27 Table 5-7. This effectively reduces the number of containment-related SAMAs to eight.

28
29 Duke also considered potential alternatives to reduce containment failure modes of concern for
30 McGuire. These alternatives included nine containment-related improvements evaluated as
31 part of the Staff's assessment of SAMDAs for Watts Bar (NRC 1995a) and five containment-
32 related improvements (e.g., procedures for reactor coolant system depressurization and
33 procedures to cope with and reduce induced SGTR) derived from the Staff's generic report on
34 the individual plant examination program (NRC 1997c). Duke eliminated those alternatives that
35 are either (1) already implemented at McGuire or (2) not applicable to the McGuire containment
36 design. Based on the screening, Duke designated nine of the containment-related SAMAs for
37 further study. The list of the potential enhancements to improve containment performance is
38 presented in Table 5-7.

Table 5-6. SAMA Cost/Benefit Screening Analysis—SAMAs That Reduce CDF

Potential Alternative	Sequences/Failures Addressed	Risk Reduction		Total Benefit	Cost of Enhancement
		CDF ^(a)	Population Dose ^(b) (person-rem ^(c))		
Man standby shutdown facility (SSF) 24 hours/day with trained operator	Loss of service water (RN), failure of operators to align safe shutdown (SS) system for operation, filter (Standby Makeup Pump) restricts flow, failure to align reactor vessel (RV) cooling/other Unit RN Vital instrumentation and control (I&C) Fire causes a Loss of RN, failure of operators to align SS system for operation, failure to use other Unit or remote control during fire Loss of 4160V Essential Bus and failure to align SS system for operation <u>AND</u> Tornado causes LOOP, DG 1A and 1B fail to fun, operators fail to initiate SS system operation	1.1E-5	3.2	\$380,000	>\$2.5M ^(d)
(a) Total CDF = 4.9E-5 per year (b) Total population dose = 13.5 person-rem per year (c) One person-Sv = 100 person-rem (d) Cost estimates for manning the standby shutdown system apply on a per-site rather than a per-unit basis. To provide a consistent basis for comparison with the estimated benefits (which are per unit), the estimated site costs were divided by two.					

Table 5-6. cont'd

Potential Alternative	Sequences/Failures Addressed	Risk Reduction		Total Benefit	Cost of Enhancement
		CDF ^(a)	Population Dose ^(b) (person-rem ^(c))		
Install automatic swap over to high-pressure recirculation	LOCA cut sets with failure of operators to establish high pressure recirculation	0	0.4	\$291,000	>\$1M
Install automatic swap to reactor vessel cooling/other unit RN system upon loss of RN	Loss of RN, failure of operators to align SS System for operation, filter (Standby Makeup Pump) restricts flow, failure to align RV cooling/other Unit RN	0	1.2	\$275,000	>\$1M
Install third diesel generator	Tornado causes LOOP, DG 1A and 1B fail, and operators fail to initiate SS System operation	0	3.1	\$304,000	>\$2M
Install automatic swap to other unit	Vital I&C Fire causes a Loss of RN, failure of operators to align SS system for operation, failure to use other Unit or remote control during fire	0	1.1	\$106,000	>\$1M
Increase test frequency of standby makeup pump flow path (currently tested quarterly)	Loss of RN, failure of operators to align SS Sys. for operation, filter (Standby Makeup Pump) restricts flow, failure to align RV cooling/other Unit RN	0	0.5	\$62,000	>\$0.4M
Replace reactor vessel with stronger vessel	Failure of reactor pressure vessel with failure to prevent core damage following a reactor pressure vessel breach	0	<0.1	\$30,000	>\$1M
(a) Total CDF = 4.9E-5 per year (b) Total population dose = 13.5 person-rem per year (c) One person-Sv = 100 person-rem					

Table 5-7. SAMA Cost/Benefit Screening Analysis—SAMAs That Improve Containment Performance

Potential Alternative	Risk Reduction		Total Benefit	Cost of Enhancement
	CDF	Population Dose (person-rem ^(a))		
Install independent containment spray system	NA	10.8	\$349,000 ^(b)	>\$1M
Install filtered containment vent system	NA	10.8	\$349,000 ^(b)	>\$1M
Install back-up power to igniters and install back-up power to air return fans	NA	10.8	\$349,000 ^(b)	\$270 K ^(c)
Install containment inerting system	NA	10.8	\$349,000 ^(b)	>\$1M
Install additional containment bypass instrumentation	NA	2.6	\$84,000	>\$1M
Add independent source of feedwater to reduce induced SGTR	NA	< 0.1	< \$3,200	>\$1M
Install reactor cavity flooding system	NA	5.6	\$181,000	>\$1M
Install core retention device	NA	< 0.1	< \$3,200	>\$1M
(a) One person-Sv = 100 person-rem (b) Total benefit based on eliminating all early and late containment failures (c) Cost estimates for back-up were provided on a per site rather than per unit basis. In order to provide a consistent basis for comparison with the estimated benefits (which are per unit), the estimated site costs were divided by two.				

5.2.3.2 Staff Evaluation

It should be noted that Duke has made extensive use of PRA methods to gain insights regarding severe accidents at McGuire. Risk insights from various McGuire risk assessments have been identified and implemented to improve both the design and operation of the plant. For example, using the IPE process, Duke (1) modified procedures to better cope with a loss of nuclear service water event and to better prioritize operator actions in a loss of alternating current (ac) power event; (2) added procedures to exercise the nuclear service water cross-connect valves between Unit 1 and 2 during each refueling outage; (3) fitted expansion joints in the nuclear service water piping located in the auxiliary feedwater pump room with a collar to limit the leak rate; (4) made a number of changes to enhance the reliability of the Emergency Diesel Generator System; (5) performed training exercises to demonstrate that the operators

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1 can activate the standby shutdown facility (SSF) within 10 minutes; and (6) expanded the
2 refueling water storage tank (FWST) level instrumentation span to the full range to reduce the
3 potential for operator error during switchover to sump recirculation. Examples of plant
4 improvements being planned for implementation by Duke based on IPEEE findings include (1)
5 adding spacers between the Unit 1 DG batteries and racks; (2) adding grout between
6 component cooling heat exchangers saddle base and concrete curb; (3) trimming the grating
7 around the steam vent valves; (4) replacing some missing bolts on the Unit 2 upper surge
8 tanks; and (5) adding some additional procedural guidelines to secure movable equipment and
9 structures to prevent potential seismic interactions. The implementation of such improvements
10 reduced the risk associated with the major contributors identified by the McGuire PRA and
11 contributed to the reduced number of candidate SAMAs identified as part of Duke's application
12 for license renewal.

13
14 Duke's effort to identify potential SAMAs focused on areas found to be risk-significant in the
15 McGuire PRA. The SAMAs listed generally coincide with accident categories that are dominant
16 CDF contributors or with issues that tend to have a large impact on a number of accident
17 sequences at McGuire. Duke made a reasonable effort to use the McGuire PRA to search for
18 potential SAMAs and to review insights from other plant-specific risk studies and previous
19 SAMA analyses for potential applicability to McGuire. The staff reviewed the set of potential
20 enhancements considered in Duke's SAMA identification process. These include
21 improvements oriented toward reducing the CDF and risk from major contributors specific to
22 McGuire and improvements identified in the previous SAMDA review for Watts Bar (NRC
23 1995a) that would be applicable to McGuire.

24
25 The staff notes that most of the SAMAs involve major modifications and significant costs and
26 that less expensive design improvements and procedure changes could conceivably provide
27 similar levels of risk reduction. The staff requested additional information (NRC 2001) from
28 Duke on less expensive alternatives that would yield similar benefits. In response, Duke
29 provided additional information on alternative power to hydrogen igniters for SBO and passive
30 autocatalytic recombiners (PARs) as an alternative to igniters. Duke also provided an estimate
31 of the cost to install a dedicated line from the Cowan's Ford hydroelectric station, as an
32 alternative source of ac power. This information was responsive to the staff's requests and
33 provided additional depth to the SAMAs considered. These additional alternatives are further
34 evaluated, along with the other SAMAs, in the sections that follow.

35
36 The staff concludes that Duke has used a systematic process for identifying potential design
37 improvements for McGuire and that the set of potential design improvements identified by Duke
38 is reasonably comprehensive and, therefore, acceptable.
39

5.2.4 Risk Reduction Potential of Design Improvements

Section 4.3 of Attachment K to the ER describes the process used by Duke to determine the risk reduction potential for each enhancement.

For each seismic initiator cut set, Duke calculated the associated offsite risk based on the population dose and CDF for the PDSs attributable to the seismic initiator. Implementation of the plant enhancement was assumed to completely eliminate the seismic risk associated with the cut set. For each (non-seismic) sequence/enhancement, Duke evaluated the severe accident sequences. In general, where an alternative impacted more than one severe accident sequence, Duke determined the cumulative risk reduction achievable by each SAMA. This was performed by identifying which basic events in the cut sets would be affected by the implementation of the particular SAMA and assuming that the implementation of the SAMA would eliminate the basic event. For each containment-related improvement, Duke assumed that all of the population dose associated with the release categories impacted by the SAMA would be eliminated. For those alternatives that benefit more than one containment failure mode (i.e., independent containment spray system, filtered containment vent, back-up power to igniters and air return fans, containment inerting system, and reactor cavity flooding system), the total population dose for all affected failure modes was assumed to be completely eliminated by implementing the alternative. For example, installation of a standpipe in containment for reactor cavity flooding, which could reduce the likelihood of both early containment failure associated with reactor vessel breach and late containment failure due to basemat melt-through, was assumed to completely eliminate the associated early and late containment failures.

In responses to follow-up RAIs (Duke 2002), Duke noted that the risk reduction estimates had changed in some instances when the PRA was updated to Revision 3. The Revision 3 CDF results are summarized in Section 5.2.2.2. One significant change was an increase in the CDF for SGTR events. According to Duke, this change yielded an estimated increase in population dose of approximately 0.04 person-Sv (4 person-rem). Duke reassessed the benefits of completely eliminating SGTR based on this new information, and calculated a maximum benefit of \$101,000 (present worth for the 20-year license renewal period). It is Duke's position that it is unlikely that a cost-beneficial alternative could be implemented to further reduce the SGTR risk based on such a low benefit estimate. The staff concurs with this assessment. Use of the PRA Revision 3 CDF estimates in lieu of the PRA Revision 2 CDF values would not appear to introduce any other significant changes to the risk profile that would make any of the other candidate SAMAs more cost-beneficial and might make some SAMAs less cost-beneficial, particularly SAMAs related to SBO events.

The staff questioned (NRC 2001) Duke regarding why the SAMA involving addition of a third DG was estimated to provide only a small (about 36 percent) reduction in the CDF for SBO sequences. Duke indicated that the risk reduction was based on eliminating all failures to start, failures to run, and common-cause failures of the existing two diesels. However, it was

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1 assumed that the third diesel would not be of seismically qualified; therefore, it would not be
2 effective in seismic events. Because seismic events account for approximately half of the SBO
3 CDF, the limited risk reduction estimated for the third diesel appears reasonable. Duke also
4 considered the additional benefit if the third diesel were seismically qualified, similar to the
5 existing DGs. Duke estimated that an additional reduction in CDF of about $1.3\text{E-}6$ per year
6 would be achieved by eliminating all random failures of DGs in seismic events. This risk
7 reduction is limited because the seismic results are dominated by seismic failures in the 4-kV
8 power system for which improving DG availability provides no benefit. The Staff concludes that
9 Duke's risk reduction estimates for this SAMA are reasonable.

10
11 An estimate of the risk reduction for the SAMA involving installation of a dedicated power line
12 from the Cowan's Ford hydroelectric station was not provided in Duke's RAI response.
13 However, the risk reduction would be comparable to that for adding a third DG, because the
14 seismic fragility of the hydroelectric unit is expected to be similar to that for the seismically
15 qualified DGs.

16
17 The Staff notes that Duke evaluated the risk reduction potential for each SAMA, including the
18 dedicated power line, in a bounding fashion. Each SAMA was assumed to completely eliminate
19 all sequences that the specific enhancement was intended to address; therefore, the benefits
20 are generally overestimated and conservative. The Staff also notes that use of the PRA
21 Revision 3 CDF estimates in lieu of the PRA Revision 2 CDF values would not appear to
22 introduce any significant changes to the risk profile that would make any of the candidate
23 SAMAs cost-beneficial, including SAMAs related to SGTR events. Accordingly, the Staff based
24 its estimates of averted risk for the various SAMAs on Duke's risk reduction estimates.

25 26 **5.2.5 Cost Impacts of Candidate Design Improvements**

27
28 Duke's estimated costs for each potential design enhancement are provided in Table 4-2 and
29 Section 5.3 of Attachment K to the ER. For most of the SAMAs, Duke estimated the cost of
30 implementation to be greater than \$1 million based on cost estimates developed in previous
31 industry studies. For two SAMAs, Duke developed plant-specific cost estimates because there
32 was no readily available information on the estimated cost to implement similar alternatives and
33 because the basic events associated with these alternatives were found to have a high
34 importance in the McGuire PRA. These SAMAs involve (1) installing a third DG, and (2)
35 increasing the test frequency of the standby makeup pump flow path. The costs to implement
36 these SAMAs were estimated to be on the order of \$2M and \$435,000, respectively. Because
37 the benefits of the potential SAMAs were significantly less than their estimated implementation
38 costs (by a factor of three or more), none of the cost estimates were further refined.
39 Specifically, the benefit of adding a third DG was about \$304,000 while the benefit of increasing
40 the test frequency was about \$62,000 (see Table 5-6).
41

The staff compared Duke's cost estimates with estimates developed elsewhere for similar improvements, including estimates developed as part of the evaluation of SAMDAs for operating reactors and advanced light-water reactors (LWRs). The staff notes that Duke's estimated implementation costs of \$1 million dollars or greater are consistent with the values reported in previous analyses for major hardware changes of similar scope and are not unreasonable for the SAMAs under consideration, given that these enhancements involve major hardware changes and impact safety-related systems. For example, Duke estimated the cost to install a third DG to be approximately \$2M; this value is less than the cost estimates reported in previous SAMDA analyses for a similar design change.

Duke's estimate of the cost to install a dedicated line from the Cowan's Ford hydroelectric station as an alternate source of ac power also appears reasonable. This line would be buried to eliminate weather-related common-cause failures. The estimated cost (\$3M) is comparable to the cost estimate provided by Dominion Power (NRC 2002c) for a similar modification at the Surry Nuclear Power Station (\$2 million to \$5 million), but is far greater than the calculated benefit of \$300K for McGuire.

The Staff questioned Duke regarding the costs of less expensive alternatives that could offer similar risk reduction benefits, particularly with regard to hydrogen control in SBO events. In a January 31, 2002, response to Staff RAIs (Duke 2002), Duke provided additional information on the costs associated with installing a passive hydrogen control system based on the use of PARs in lieu of the present ac-dependent hydrogen igniters, and the costs of powering a subset of the current hydrogen igniters from a back-up generator. For scoping purposes, Duke provided supplementary information regarding the cost of back-up power to the igniters and air return fans in response to a follow-up RAI (NRC 2002a).

Duke's estimate of the cost to establish a capability to power a subset of igniters from a back-up generator was \$205,000 for the site. This modification, as defined by Duke, would involve pre-staging a single, dedicated generator outdoors on a concrete pad (for ventilation and exhaust considerations) and supplying the necessary power cables and circuit breakers to enable connection to the igniter branch circuits in either unit. The breakdown of this cost is: \$5,000 for engineering, \$50,000 for materials, \$110,000 for installation labor, and \$40,000 for maintenance and operation. This cost estimate does not include an enclosure, tornado protection for the generator, or any seismic design. When one air return fan is added to this estimate, the combined cost is \$540,000. The breakdown of this cost is: \$50,000 for engineering, \$210,000 for materials \$240,000 for installation labor, and \$40,000 for maintenance and operation. Duke points out there will be additional costs not included in these estimates. In order to provide a consistent basis for comparison with the estimated benefits (which are per unit), the above costs were divided by two.

The staff requested additional information on PARs, because PARs are to be installed in French PWRs by 2007 to mitigate the consequences of hydrogen combustion events. In response (Duke 2002), Duke estimated that the installation of PARs would cost more than

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1 \$750,000 per unit, which is well above the estimated benefit (see Table 5-8, Section 5.2.6.2).
2 This cost estimate is consistent with independent staff cost estimates for installing PARs. Duke
3 further noted that providing electric power to hydrogen igniters during a SBO or installing PARs
4 will not be effective without also powering at least one of the containment air return fans and
5 that this will further increase the cost of these options.

6
7 The staff asked for further information on the basis for the greater than \$1M cost estimate for
8 two other SAMAs: (1) install automatic swap-over to high pressure recirculation, and (2) install
9 automatic swap to reactor vessel cooling or the other unit's service water system upon loss of
10 the service water system. Duke (NRC 2002a) referenced NUREG-0498, Supp. 1 (NRC 1995a),
11 which estimated a cost of about \$2.1M for a similar alternative, i.e., "automate the alignment of
12 emergency core cooling system (ECCS) recirculation to the high-pressure charging and safety
13 injection pumps." This would reduce the potential for related human errors made during
14 manual realignment. This cost estimate applies to both of these candidate SAMAs and is
15 considerably higher than the estimated averted risk benefit for McGuire of about \$275,000 to
16 \$291,000. (Benefits are discussed further in Section 5.2.6.)

17
18 The staff concludes that the cost estimates provided by Duke are reasonable and adequate for
19 the purposes of this SAMA evaluation. As noted in Section 5.2.6.2, further attention will be
20 placed on the costs associated with SBO-related plant improvements by the NRC as part of the
21 resolution of Generic Safety Issue 189 - Susceptibility of Ice Condenser and Mark III
22 Containments to Early Failure from Hydrogen Combustion During a Severe Accident
23 (NRC 2002b).

24 25 **5.2.6 Cost-Benefit Comparison**

26
27 The cost-benefit comparison as evaluated by Duke and the staff evaluation of the cost-benefit
28 analysis are described in the following sections.

29 30 **5.2.6.1 Duke Evaluation**

31
32 In the analysis provided in the McGuire ER, Duke did not include the following factors in its
33 cost-benefit evaluation: replacement power costs for SAMAs that have the potential to reduce
34 CDF and averted offsite property damage costs for SAMAs that have the potential to improve
35 containment performance. In view of the significant impact of these averted costs on the
36 estimated benefit for a SAMA, the staff requested that Duke include these factors in the cost-
37 benefit analysis for each affected SAMA. In response to the RAI (Duke 2002), Duke updated
38 the benefit estimates to include averted replacement power costs (ARPC) and averted offsite
39 property damage costs (AOC).
40

The methodology used by Duke was based primarily on NRC's guidance for performing cost-benefit analysis in NUREG/BR-0184, *Regulatory Analysis Technical Evaluation Handbook* (NRC 1997b). The guidance involves determining the net value for each SAMA according to the following formula:

$$\text{Net Value} = (\$APE + \$AOC + \$AOE + \$AOSC) - COE$$

where \$APE = present value of averted public exposure (\$)
 \$AOC = present value of averted offsite property damage costs (\$)
 \$AOE = present value of averted occupational exposure costs (\$)
 \$AOSC = present value of averted onsite costs (\$)
 COE = cost of enhancement (\$).

If the net value of a SAMA is negative, the cost of implementing the SAMA is larger than the benefit associated with the SAMA and it is not considered cost-beneficial. Duke's derivation of each of the associated costs is summarized below.

Averted Public Exposure (APE) Costs

The APE costs were calculated using the following formula:

$$\begin{aligned} \text{APE} = & \text{Annual reduction in public exposure } (\Delta \text{person-rem/reactor year}) \\ & \times \text{monetary equivalent of unit dose } (\$2000 \text{ per person-rem}) \\ & \times \text{present value conversion factor } (10.76 \text{ based on a 20-year period with a 7} \\ & \text{percent discount rate}). \end{aligned}$$

As stated in NUREG/BR-0184 (NRC 1997b), it is important to note that the monetary value of the public health risk after discounting does not represent the expected reduction in public health risk due to a single accident. Rather, it is the present value of a stream of potential losses extending over the remaining lifetime (in this case, the renewal period) of the facility. Thus, it reflects the expected annual loss due to a single accident, the possibility that such an accident could occur at any time over the renewal period, and the effect of discounting these potential future losses to present value. Duke used the following expression when calculating the APE for the 20-year license renewal period:

$$\text{APE} = \$2.20\text{E}+04 \times (\text{Change in public exposure}).$$

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Averted Offsite Property Damage Costs (AOC)

For SAMAs that reduce CDF, the AOCs were calculated using the following formula:

$$\begin{aligned} \text{AOC} = & \text{Annual CDF reduction} \\ & \times \text{offsite economic costs associated with a severe accident (on a per-event basis)} \\ & \times \text{present value conversion factor.} \end{aligned}$$

Duke derived the values for averted offsite property damage costs based on information provided in Section 5.7.5 of NUREG/BR-0184 (NRC 1997b). A discount factor of 7 percent and a 4 percent rate of inflation were used. Duke used the following expression when calculating the AOC for the 20-year license renewal period:

$$\text{AOC} = \$3.92\text{E}+09 \times (\text{Change in annual CDF}).$$

Originally, as part of the ER, Duke did not include the AOC for containment-related SAMAs. In response to staff RAIs, Duke incorporated AOC as follows (Duke 2002).

For containment-related SAMAs (which impact population dose but not CDF), Duke estimated the combined AOC and averted public exposure costs (APE) based on a conversion factor of \$3000/person-rem, which was attributed to NUREG/CR-6349 (NRC 1995b). Duke used the following expression when calculating these costs (for containment-related SAMAs) for the 20-year license renewal period:

$$\text{AOC} + \text{APE} = \$3.23\text{E}+04 \times (\text{Change in public exposure}).$$

Averted Occupational Exposure (AOE) Costs

The AOE costs were calculated using the following formula:

$$\begin{aligned} \text{AOE} = & \text{Annual CDF reduction} \\ & \times \text{occupational exposure per core damage event} \\ & \times \text{monetary equivalent of unit dose} \\ & \times \text{present value conversion factor.} \end{aligned}$$

Duke derived the values for averted occupational exposure based on information provided in Section 5.7.3 of NUREG/BR-0184 (NRC 1997b). Best-estimate values provided for immediate occupational dose [33 person-Sv (3300 person-rem)] and long-term occupational dose [200 person-Sv (20,000 person-rem) over a 10-year cleanup period] were used. The present value of these doses was calculated using the equations provided in NUREG/BR-0184 in conjunction with a monetary equivalent of unit dose of \$2000 per person-rem, a discount rate of 7 percent,

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and a time period of 20 years to represent the license-renewal period. Duke used the following expression when calculating the AOE for the 20-year license renewal period:

$$AOE = \$3.81E+08 \times (\text{Change in annual CDF}).$$

Averted Onsite Costs (AOSC) (Not Including Replacement Power Costs)

The AOSCs, as calculated by Duke, include averted cleanup and decontamination costs. NUREG/BR-0184, Section 5.7.6.2, states that long-term replacement power costs must also be considered (NRC 1997b). Duke did not include this cost in the ER. However, Duke did add this cost in the responses (Duke 2002) to the staff's RAIs.

Averted cleanup and decontamination costs (ACC) were calculated using the following formula:

$$\begin{aligned} \text{ACC} = & \text{Annual CDF reduction} \\ & \times \text{present value of cleanup costs per core damage event} \\ & \times \text{present value conversion factor.} \end{aligned}$$

The total cost of cleanup and decontamination subsequent to a severe accident is estimated in NUREG/BR-0184 (NRC 1997b) as \$1.5E+09 (undiscounted). This value was converted to present costs over a 10-year cleanup period and integrated over the term of the proposed license extension. Duke used the following expression when calculating the ACC for the 20-year license renewal period:

$$ACC = \$1.18E+10 \times (\text{Change in annual CDF}).$$

Averted Power Replacement Cost (ARPC)

The Duke estimate of the annual power replacement cost for McGuire is based on an assumed discount rate of 7 percent for the 20-year license renewal period.

The estimated present power replacement costs of a severe accident occurring in each year of the license renewal period is given by (equation from NUREG/BR-0184):

$$PV_{RP} = [\$1.2E+08/0.07][1 - \exp(-0.07 * 20)]^2$$

$$PV_{RP} = \$9.73E+08$$

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Then, to estimate the net present value of power replacement over the 20-year license renewal (equation from NUREG/BR-0184, p. 5.44):

$$U_{RP} = [PV_{RP}/0.07][1 - \exp(-0.07 * 20)]^2$$

$$U_{RP} = \$7.89E+09$$

Averted Power Replacement Cost (APRC) = $U_{RP} * (\text{Change in annual CDF})$.

Because the averted power replacement cost from the NUREG is in 1990 dollars, an assumption is made to include a 4 percent inflation rate over 11 years to bring the value into 2001 dollars; therefore,

$$APRC = \$1.21E+10 * (\text{Change in annual CDF}).$$

Duke Results

The total benefit associated with each of the 15 SAMAs evaluated by Duke (7 that reduce CDF and 8 that improve containment performance) is provided in Tables 5-6 and 5-7. One of the SAMAs has a positive net value (i.e., the total benefit is greater than the cost of the enhancement). All of the remaining SAMAs have a negative net value, even given the bounding risk-reduction benefits inherent in these estimates.

5.2.6.2 Staff Evaluation

The cost-benefit analysis provided by Duke (Duke 2001, 2002) was based primarily on NRC's *Regulatory Analysis Technical Evaluation Handbook* (NRC 1997b). In the original ER, Duke did not include averted replacement power costs for SAMAs that reduce CDF or averted offsite property damage costs for SAMAs that improve containment performance. However, the impact of these factors was included in supplemental analyses provided by Duke in response to the staff's RAI's (Duke 2002; NRC 2002a). The averted replacement power costs were assessed appropriately and the values calculated by Duke are consistent with independent staff assessments.

Duke used a conversion factor of \$3,000/person-rem to determine the averted offsite property damage and averted public exposure costs. This effectively assumes a \$1,000/person-rem conversion factor as a surrogate for averted offsite property damage, in addition to the accepted \$2,000/person-rem conversion factor for averted offsite public exposure costs. Because offsite property damage costs are plant-and site-specific, it would be more consistent with standard practice to actually calculate the property damage using the MACCS code. Nevertheless, the averted offsite costs values (for health effects and property damage) calculated by Duke provide reasonably good agreement with typical site values and are

1 acceptable for purposes of estimating the value of containment-related SAMAs. Inclusion of
2 averted replacement power and offsite property damage costs did not result in identification of
3 any additional cost-beneficial SAMAs, and would not call into question Duke's decision to
4 eliminate seismic SAMAs from consideration, given the large costs associated with seismic
5 SAMAs.

6
7 For most of the candidate SAMAs, the staff agrees with Duke that the SAMAs would clearly not
8 be cost-beneficial because they have costs that are substantially (typically a factor of three or
9 more) higher than the dollar equivalent of the associated benefits. This difference is considered
10 to provide ample margin to cover uncertainties in the risk and cost estimates because estimates
11 for these factors were generally evaluated in a conservative manner. This is true even when
12 considering the 3 percent versus 7 percent discount rate sensitivity case or the use of a 40-year
13 versus 20-year time period. However, the cost-benefit analyses for the some of the SAMAs
14 related to hydrogen control in SBO events have benefits that are similar in magnitude to the
15 costs. The frequency of SBO events for McGuire account for 47 percent of the total CDF of
16 $4.9\text{E-}5$ per year based on Revision 2 of the PRA and 22 percent of the total CDF of $4.6\text{E-}5$ per
17 year based on Revision 3 of the PRA. Also, ice condenser containments have a higher degree
18 of vulnerability to hydrogen combustion in SBO events, as described in NUREG/CR-6427
19 (NRC 2000).

20
21 NUREG/CR-6427 studied the direct containment heating (DCH) issue for plants with ice
22 condenser containments (NRC 2000) and found that early containment failure is dominated by
23 hydrogen combustion events rather than DCH events, and that no ice condenser plant is
24 inherently robust to all credible DCH or hydrogen combustion events in station blackout. The
25 study concluded that all plants, especially McGuire, would benefit from reducing SBO frequency
26 or from providing some means of hydrogen control that is effective in SBO events. In light of
27 the issues raised in NUREG/CR-6427 concerning the likelihood of early containment failure in
28 SBO events, the staff requested Duke to provide a reevaluation of the benefits associated with
29 the hydrogen control measures (install back-up power to igniters and air return fans) assuming
30 a containment response consistent with the findings in NUREG/CR-6427 (i.e., using the
31 containment failure probabilities for DCH and non-DCH events reported in the study, in place of
32 the conditional failure probabilities implicit in the baseline PRA). Under these assumptions,
33 Duke estimated that the averted population dose risk from eliminating early containment
34 failures would rise from a base case value of 0.055 person-Sv (5.5 person-rem) per year to
35 0.21 person-Sv (21 person-rem) per year. The benefit values based on use of the NUREG/CR-
36 6427 containment failure probability for McGuire are reported in Table 5-8. Also shown are the
37 benefits values for the sensitivity cases involving use of a 3 percent discount rate compared to
38 a 7 percent discount rate in the base case and use of the SBO CDF estimates from Revision 3
39 of the PRA rather than Revision 2. All of the values in Table 5-8 include averted offsite property
40 damage.

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Table 5-8. Sensitivity Results for Hydrogen Control SAMAs (all benefits based on eliminating early failures only)

SAMA	Estimated Cost (per unit)	Estimated Benefits for Hydrogen Control SAMAs Under Various Assumptions			
		Based on Revision 2 of the PRA	Based on conditional containment failure probabilities from NUREG/CR-6427	Based on a 3% discount rate compared to a 7% discount rate in the base case	Based on SBO values from Revision 3 of the PRA
Back-up power to igniters & air return fans	\$270,000 ^(a)	\$178,000	\$678,000	\$248,000	\$76,000
PARs	\$750,000	\$178,000	\$678,000	\$248,000	\$76,000
Back-up power to igniters only	\$102,500 ^(a)	Duke: no benefit, since air-return fans are needed	Duke: no benefit, since air-return fans are needed	Duke: no benefit, since air-return fans are needed	Duke: no benefit, since air-return fans are needed
• Cost estimates for back-up power were provided on a per-site rather than a per-unit basis. To provide a consistent basis for comparison with the estimated benefits (which are per unit), the estimated site costs were divided by two.					

A number of points are worth noting regarding the Duke base case results and these sensitivity assessments:

- Not all early and late releases can be eliminated by providing hydrogen control. For example, late failures due to long-term containment over-pressure could still occur. Also, the non-safety related, non-seismic back-up power source may not be available in large seismic and tornado events, if it is not designed to withstand such events. An upper bound estimate can be provided by assuming that all containment failures—early and late—would be eliminated. More realistically, most of the early and some of the late releases would be eliminated. The assumption that hydrogen control would eliminate all early failures is considered to provide a reasonable estimate of the risk reduction benefit. Accordingly, the estimated benefits shown in Table 5-8 are based on eliminating all early containment failures.

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- It is Duke's position that powering the igniters without also powering the air-return fans would not achieve effective hydrogen control. According to Duke, in order to realize the stated benefits, the air-return fans must also have a back-up power source. More than half of the cost of the SAMA to provide back-up power to igniters and air-return fans comes from powering the fans. Based on available technical information, it is not clear that operation of an air-return fan is necessary to provide effective hydrogen control. If only the igniters need to be powered during SBO, a less expensive option of powering a subset of igniters from a back-up generator, addressed by Duke in responses to RAIs (Duke 2002; NRC 2002a), is within the range of averted risk benefits and would warrant further consideration.
- If a 3 percent discount rate is assumed in contrast to the 7 percent discount rate assumed in the base case analysis, the benefits are similar in magnitude to the costs, even when including back-up power to the air-return fan. This further supports the position that the benefits are large and that a hydrogen-related SAMA may be cost-beneficial.
- The effect of implementing the SAMA in the near term rather than delaying implementation until the start of the license renewal period (i.e., use of a 40-year rather than a 20-year period in the value impact analyses) is bounded by the sensitivity study that assumed a 3 percent discount rate.
- The Revision 3 PRA results would reduce the averted risk benefits by about half. While this is a substantial reduction, it does not eliminate the generic concern that the benefits of additional hydrogen control are large.

The NRC has recognized that ice condenser containments like McGuire's are vulnerable to hydrogen burns in the absence of power to the hydrogen ignitor system. This issue is sufficiently important for all PWRs with ice condenser containments that NRC has made the issue a Generic Safety Issue (GSI), GSI-189 - Susceptibility of Ice Condenser and Mark III Containments to Early Failure from Hydrogen Combustion During a Severe Accident (NRC 2002b). As part of the resolution of GSI-189, NRC is evaluating potential improvements to hydrogen control provisions in ice condenser plants to reduce their vulnerability to hydrogen-related containment failures in SBO. This will include an assessment of the costs and benefits of supplying igniters from alternate power sources, such as a back-up generator, as well as containment analyses to establish whether air-return fans also need an ac-independent power source, as part of this modification. The need for plant design and procedural changes will be resolved as part of GSI-189 and addressed for McGuire and other ice condenser plants as a current operating license issue.

5.2.7 Conclusions

Duke completed a comprehensive effort to identify and evaluate potential cost-beneficial plant enhancements to reduce the risk associated with severe accidents at McGuire. As a result of this assessment, Duke concluded that no additional mitigation alternatives are cost-beneficial and warrant implementation at McGuire.

Based on its review of SAMAs for McGuire, the Staff concurs that none of the candidate SAMAs are cost-beneficial with the possible exception of one SAMA related to hydrogen control in SBO events. This conclusion is consistent with the low level of risk indicated in the McGuire PRA and the fact that Duke has already implemented numerous plant improvements identified from previous plant-specific risk studies. Duke's position is that SAMAs that provide hydrogen control in SBO events are not cost-effective because back-up power would also need to be supplied to the air-return fans from ac-independent power sources in order to ensure mixing of the containment atmosphere; the cost of powering both the igniters and the air-return fans would exceed the expected benefit. However, based on available technical information, it is not clear that operation of an air-return fan is necessary to provide effective hydrogen control. If only the igniters need to be powered during SBO, a less-expensive option of powering a subset of igniters from a back-up generator, addressed by Duke in responses to RAIs (Duke 2002; NRC 2002a), is within the range of averted risk benefits and would warrant further consideration. Even if air-return fans are judged to be necessary to ensure effective hydrogen control in SBOs, the results of sensitivity studies suggest that this combined SAMA might also be cost-beneficial.

The staff concludes that one of the SAMAs related to hydrogen control in SBO sequences (supplying existing hydrogen igniters with back-up power from an independent power source during SBO events) is cost-beneficial under certain assumptions, which are being examined in connection with resolution of GSI-189. However, this SAMA does not relate to adequately managing the effects of aging during the period of extended operation. Therefore, it need not be implemented as part of license renewal pursuant to 10 CFR Part 54. The staff has recognized hydrogen control in SBO sequences as an operating license issue for all ice condenser plants. The need for plant design and procedural changes will be resolved as part of GSI-189 and addressed for McGuire and other ice condenser plants as a current operating license issue.

5.3 References

10 CFR 50. Code of Federal Regulations, Title 10, *Energy*, Part 50, "Domestic Licensing of Production and Utilization Facilities."

10 CFR 51. Code of Federal Regulations, Title 10, *Energy*, Part 51, "Environmental Protection Regulations for Domestic Licensing and Related Regulatory Functions."

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1 10 CFR Part 51, Subpart A, Appendix B, Table B-1, "Summary of Findings on NEPA Issues for
2 License Renewal of Nuclear Power Plants."

3
4 10 CFR 54. Code of Federal Regulations, Title 10, *Energy*, Part 54, "Requirements for Renewal
5 of Operating Licenses for Nuclear Power Plants."

6
7 10 CFR 100. Code of Federal Regulations, Title 10, *Energy*, Part 100, "Reactor Site Criteria."

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9 Duke Power Company (Duke Power). 1991. Letter from T. C. McMeekin, DPC to NRC. Subject:
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11 dated November 4, 1991.

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13 Duke Power Company (Duke Power). 1994. Letter from T. C. McMeekin, DPC to NRC. Subject:
14 Individual Plant Examination of External Events (IPEEE) Submittal, McGuire Nuclear Station,
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17 Duke Energy Corporation (Duke). 1998. Probabilistic Risk Assessment, Individual Plant
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21 *Renewal Stage, McGuire Nuclear Station Units 1 and 2*. Charlotte, North Carolina.

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23 Duke Energy Corporation (Duke). 2002. Letter from M. S. Tuckman of Duke Energy Corporation
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27 January 31, 2002.

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32 U.S. Nuclear Regulatory Commission (NRC). 1990. *Severe Accident Risks - An Assessment for*
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38 U.S. Nuclear Regulatory Commission (NRC). 1994. Letter from V. Nerses (NRC) to T. C.
39 McMeekin (Duke Power Company), Subject: Staff Evaluation of the McGuire Nuclear Station,
40 Units 1 and 2, Individual Plant Examination - Internal Events Only, June 30, 1994.

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1 U.S. Nuclear Regulatory Commission (NRC). 1995a. *Final Environmental Statement Related to*
2 *the Operation of Watts Bar Nuclear Plant Units 1 and 2*. NUREG-0498, Supplement 1,
3 Washington, D.C.

4
5 U.S. Nuclear Regulatory Commission (NRC). 1995b. *Cost-Benefit Considerations in Regulatory*
6 *Analysis*. NUREG/CR-6349. U.S. Nuclear Regulatory Commission, Washington, D.C.

7
8 U.S. Nuclear Regulatory Commission (NRC). 1996. *Generic Environmental Impact Statement*
9 *for License Renewal of Nuclear Plants*. NUREG-1437, Volumes 1 and 2, Washington, D.C.

10
11 U.S. Nuclear Regulatory Commission (NRC). 1997a. *SECPOP90: Sector Population, Land*
12 *Fraction, and Economic Estimation Program*. NUREG/CR-6525, Washington, D.C.

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14 U.S. Nuclear Regulatory Commission (NRC). 1997b. *Regulatory Analysis Technical Evaluation*
15 *Handbook*. NUREG/BR-0184, Washington, D.C.

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17 U.S. Nuclear Regulatory Commission (NRC). 1997c. *Individual Plant Examination Program:*
18 *Perspectives on Reactor Safety and Plant Performance*. NUREG-1560, Washington, D.C.

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20 U.S. Nuclear Regulatory Commission (NRC). 1999a. *Generic Environmental Impact Statement*
21 *for License Renewal of Nuclear Plants, Main Report*, "Section 6.3—Transportation, Table 9.1
22 Summary of findings on NEPA issues for license renewal of nuclear power plants, Final Report."
23 NUREG-1437, Volume 1, Addendum 1, Washington, D.C.

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25 U.S. Nuclear Regulatory Commission (NRC). 1999b. Letter from F. Rinaldi (NRC) to H. B.
26 Barron (Duke Energy Corporation), Subject: Review of McGuire Nuclear Station, Units 1 and 2 -
27 Individual Plant Examination of External Events Submittal, February 16, 1999.

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32 U.S. Nuclear Regulatory Commission (NRC). 2001. Letter from J. H. Wilson (NRC) to M. S.
33 Tuckman (Duke Energy Corporation), Subject: Request for Additional Information Related to the
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6.0 Environmental Impacts of the Uranium Fuel Cycle and Solid Waste Management

Environmental issues associated with the uranium fuel cycle and solid waste management were discussed in the *Generic Environmental Impact Statement for License Renewal of Nuclear Plants* (GEIS), NUREG-1437, Volumes 1 and 2 (NRC 1996, 1999a).^(a) The GEIS includes a determination of whether the analysis of the environmental issue could be applied to all plants and whether additional mitigation measures would be warranted. Issues are then assigned a Category 1 or a Category 2 designation. As set forth in the GEIS, Category 1 issues are those that meet all of the following criteria:

- (1) The environmental impacts associated with the issue have been determined to apply either to all plants or, for some issues, to plants having a specific type of cooling system or other specified plant or site characteristics.
- (2) A single significance level (i.e., SMALL, MODERATE, or LARGE) has been assigned to the impacts (except for collective offsite radiological impacts from the fuel cycle and from high-level waste [HLW] and spent fuel disposal).
- (3) Mitigation of adverse impacts associated with the issue has been considered in the analysis, and it has been determined that additional plant-specific mitigation measures are likely not to be sufficiently beneficial to warrant implementation.

For issues that meet the three Category 1 criteria, no additional plant-specific analysis is required unless new and significant information is identified.

Category 2 issues are those that do not meet one or more of the criteria of Category 1, and therefore, additional plant-specific review of these issues is required.

This chapter addresses the issues that are related to the uranium fuel cycle and solid waste management during the license renewal term that are listed in 10 CFR Part 51, Subpart A, Appendix B, and are applicable to McGuire Nuclear Station, Units 1 and 2 (McGuire). The generic potential impacts of the radiological and nonradiological environmental impacts of the uranium fuel cycle and transportation of nuclear fuel and wastes are described in detail in the GEIS, based, in part, on the generic impacts provided in 10 CFR 51.51(b), Table S-3, "Table of Uranium Fuel Cycle Environmental Data," and in 10 CFR 51.52(c), Table S-4, "Environmental Impact of Transportation of Fuel and Waste to and from One Light-Water-Cooled Nuclear

(a) The GEIS was originally issued in 1996. Addendum 1 to the GEIS was issued in 1999. Hereafter, all references to the "GEIS" include the GEIS and its Addendum 1.

Power Reactor." The GEIS also addresses the impacts from radon-222 and technetium-99. There are no Category 2 issues for the uranium fuel cycle and solid waste management.

6.1 The Uranium Fuel Cycle

Category 1 issues from 10 CFR Part 51, Subpart A, Appendix B, Table B-1 that are applicable to McGuire from the uranium fuel cycle and solid waste management are listed in Table 6-1.

Table 6-1. Category 1 Issues Applicable to the Uranium Fuel Cycle and Solid Waste Management During the Renewal Term

ISSUE- 10 CFR Part 51, Subpart A, Appendix B Table B-1	GEIS Sections
URANIUM FUEL CYCLE AND WASTE MANAGEMENT	
Offsite radiological impacts (individual effects from other than the disposal of spent fuel and high-level waste [HLW])	6.1; 6.2.1; 6.2.2.1; 6.2.2.3; 6.2.3; 6.2.4; 6.6
Offsite radiological impacts (collective effects)	6.1; 6.2.2.1; 6.2.3; 6.2.4, 6.6
Offsite radiological impacts (spent fuel and HLW)	6.1; 6.2.2.1; 6.2.3; 6.2.4, 6.6
Nonradiological impacts of the uranium fuel cycle	6.1; 6.2.2.6; 6.2.2.7; 6.2.2.8; 6.2.2.9; 6.2.3; 6.2.4; 6.6
Low-level waste storage and disposal	6.1; 6.2.2.2; 6.4.2; 6.4.3; 6.4.3.1; 6.4.3.2; 6.4.3.3; 6.4.4; 6.4.4.1; 6.4.4.2; 6.4.4.3; 6.4.4.4; 6.4.4.5; 6.4.4.5.1; 6.4.4.5.2; 6.4.4.5.3; 6.4.4.5.4; 6.4.4.6, 6.6
Mixed waste storage and disposal	6.4.5.1; 6.4.5.2; 6.4.5.3; 6.4.5.4; 6.4.5.5; 6.4.5.6; 6.4.5.6.1; 6.4.5.6.2; 6.4.5.6.3; 6.4.5.6.4, 6.6
Onsite spent fuel	6.1; 6.4.6; 6.4.6.1; 6.4.6.2; 6.4.6.3; 6.4.6.4; 6.4.6.5; 6.4.6.6; 6.4.6.7; 6.6
Nonradiological waste	6.1; 6.5; 6.5.1; 6.5.2; 6.5.3; 6.6
Transportation	6.1; 6.3.1; 6.3.2.3; 6.3.3; 6.3.4; 6.6, Addendum 1

In its environmental report (ER) (Duke 2001), Duke stated that "no new information existed for the issues that would invalidate the GEIS conclusions." No significant new information has been identified by the staff in the review process and in the staff's independent review.

Therefore, the staff concludes that there are no impacts related to these issues beyond those discussed in the GEIS. For all of those GEIS issues, the staff concluded that the impacts are SMALL except for collective offsite radiological impacts from the fuel cycle and from HLW and spent fuel disposal, as discussed below, and plant-specific mitigation measures are not likely to be sufficiently beneficial to be warranted.

A brief description of the staff review and the GEIS conclusions, as codified in Table B-1, 10 CFR 51, for each of these issues follows.

- Offsite radiological impacts (individual effects from other than the disposal of spent fuel and HLW). Based on information in the GEIS, the Commission found that

Off-site impacts of the uranium fuel cycle have been considered by the Commission in Table S-3 of this part [10 CFR 51.51(b)]. Based on information in the GEIS, impacts on individuals from radioactive gaseous and liquid releases including radon-222 and technetium-99 are small.

The staff has not identified any significant new information during its independent review of the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no offsite radiological impacts of the uranium fuel cycle during the renewal term beyond those discussed in the GEIS.

- Offsite radiological impacts (collective effects). Based on information in the GEIS, the Commission found that

The 100 year environmental dose commitment to the U.S. population from the fuel cycle, high level waste and spent fuel disposal excepted, is calculated to be about 14,800 person rem [148 person Sv], or 12 cancer fatalities, for each additional 20-year power reactor operating term. Much of this, especially the contribution of radon releases from mines and tailing piles, consists of tiny doses summed over large populations. This same dose calculation can theoretically be extended to include many tiny doses over additional thousands of years as well as doses outside the U.S. The result of such a calculation would be thousands of cancer fatalities from the fuel cycle, but this result assumes that even tiny doses have some statistical adverse health effect which will not ever be mitigated (for example no cancer cure in the next thousand years), and that these doses projected over thousands of years are meaningful. However, these assumptions are questionable. In particular, science cannot rule out the possibility that there will be no cancer fatalities from these tiny doses. For perspective, the doses are very small fractions of regulatory limits, and even smaller fractions of natural background exposure to the same populations.

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1 Nevertheless, despite all the uncertainty, some judgement as to the regulatory
2 NEPA [National Environmental Policy Act] implications of these matters should
3 be made and it makes no sense to repeat the same judgement in every case.
4 Even taking the uncertainties into account, the Commission concludes that these
5 impacts are acceptable in that these impacts would not be sufficiently large to
6 require the NEPA conclusion, for any plant, that the option of extended operation
7 under 10 CFR Part 54 should be eliminated. Accordingly, while the Commission
8 has not assigned a single level of significance for the collective effects of the fuel
9 cycle, this issue is considered Category 1.

10
11 The staff has not identified any significant new information during its independent review of
12 the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its evaluation of
13 other available information. Therefore, the staff concludes that there are no offsite
14 radiological impacts (collective effects) from the uranium fuel cycle during the renewal term
15 beyond those discussed in the GEIS.

- 16
17 • Offsite radiological impacts (spent fuel and HLW disposal). Based on information in the
18 GEIS, the Commission found that

19
20 For the high level waste and spent fuel disposal component of the fuel cycle,
21 there are no current regulatory limits for offsite releases of radionuclides for the
22 current candidate repository site. However, if we assume that limits are
23 developed along the lines of the 1995 National Academy of Sciences (NAS)
24 report, "Technical Bases for Yucca Mountain Standards," and that in accordance
25 with the Commission's Waste Confidence Decision, 10 CFR 51.23, a repository
26 can and likely will be developed at some site which will comply with such limits,
27 peak doses to virtually all individuals will be 100 millirem [1 mSv] per year or
28 less. However, while the Commission has reasonable confidence that these
29 assumptions will prove correct, there is considerable uncertainty since the limits
30 are yet to be developed, no repository application has been completed or
31 reviewed, and uncertainty is inherent in the models used to evaluate possible
32 pathways to the human environment. The NAS report indicated that 100 millirem
33 [1 mSv] per year should be considered as a starting point for limits for individual
34 doses, but notes that some measure of consensus exists among national and

1 international bodies that the limits should be a fraction of the 100 millirem [1
2 mSv] per year. The lifetime individual risk from 100 millirem [1 mSv] annual dose
3 limit is about 3×10^{-3} .
4

5 Estimating cumulative doses to populations over thousands of years is more
6 problematic. The likelihood and consequences of events that could seriously
7 compromise the integrity of a deep geologic repository were evaluated by the
8 Department of Energy in the "Final Environmental Impact Statement: Management
9 of Commercially Generated Radioactive Waste," October 1980 [DOE 1980]. The
10 evaluation estimated the 70-year whole-body dose commitment to the maximum
11 individual and to the regional population resulting from several modes of breaching a
12 reference repository in the year of closure, after 1,000 years, after 100,000 years,
13 and after 100,000,000 years. Subsequently, the NRC and other federal agencies
14 have expended considerable effort to develop models for the design and for the
15 licensing of a high level waste repository, especially for the candidate repository at
16 Yucca Mountain. More meaningful estimates of doses to population may be possible
17 in the future as more is understood about the performance of the proposed Yucca
18 Mountain repository. Such estimates would involve very great uncertainty, especially
19 with respect to cumulative population doses over thousands of years. The standard
20 proposed by the NAS is a limit on maximum individual dose. The relationship of
21 potential new regulatory requirements, based on the NAS report, and cumulative
22 population impacts has not been determined, although the report articulates the view
23 that protection of individuals will adequately protect the population for a repository at
24 Yucca Mountain. However, EPA's [Environmental Protection Agency] generic
25 repository standards in 40 CFR part 191 generally provide an indication of the order
26 of magnitude of cumulative risk to population that could result from the licensing of a
27 Yucca Mountain repository, assuming the ultimate standards will be within the range
28 of standards now under consideration. The standards in 40 CFR part 191 protect the
29 population by imposing "containment requirements" that limit the cumulative amount
30 of radioactive material released over 10,000 years. Reporting performance
31 standards that will be required by EPA are expected to result in releases and
32 associated health consequences in the range between 10 and 100 premature
33 cancer deaths with an upper limit of 1,000 premature cancer deaths world-wide for a
34 100,000 metric tonne (MTHM) repository.
35

36 Nevertheless, despite all the uncertainty, some judgement as to the regulatory
37 NEPA implications of these matters should be made and it makes no sense to
38 repeat the same judgement in every case. Even taking the uncertainties into
39 account, the Commission concludes that these impacts are acceptable in that these
40 impacts would not be sufficiently large to require the NEPA conclusion, for any plant,
41 that the option of extended operation under 10 CFR part 54 should be eliminated.
42 Accordingly, while the Commission has not assigned a single level of significance for

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the impacts of spent fuel and high level waste disposal, this issue is considered Category 1.

Since the GEIS was originally issued in 1996, the EPA has published radiation protection standards for Yucca Mountain, Nevada, at 40 CFR Part 197, "Public Health and Environmental Radiation Protection Standards for Yucca Mountain, Nevada," on June 13, 2001 (66 FR 32132). The Energy Policy Act of 1992 (42 USC 10101 et seq) directed that the NRC adopt these standards into its regulations for reviewing and licensing the repository. The NRC published its regulations at 10 CFR Part 63, "Disposal of High-Level Radioactive Wastes in a Geologic Repository at Yucca Mountain, Nevada," on November 2, 2001 (66 FR 55792). These standards include the following: (1) 0.15 mSv/year (15 mrem/year) dose limit for members of the public during the storage period prior to repository closure, (2) 0.15 mSv/year (15 mrem/year) dose limit for the reasonably maximally exposed individual for 10,000 years following disposal, (3) 0.15 mSv/year (15 mrem/year) dose limit for the reasonably maximally exposed individual as a result of a human intrusion at or before 10,000 years after disposal, and (4) a groundwater protection standard that states for 10,000 years of undisturbed performance after disposal, radioactivity in a representative volume of ground-water will not exceed (a) 0.19 Bq/L (5 pCi/L) (radium-226 and radium-228), (b) 0.56 Bq/L (15 pCi/L) (gross alpha activity), and (c) 0.04 mSv/year (4 mrem/year) to the whole body or any organ (from combined beta and photon emitting radionuclides).

On February 15, 2002, subsequent to receipt of a recommendation by Secretary Abraham, U.S. Department of Energy, the President recommended the Yucca Mountain site for the development of a repository for the geologic disposal of spent nuclear fuel and high-level nuclear waste.

This change in regulatory status does not cause the staff to change its position with respect to the impact of spent fuel and HLW disposal. The staff still considers the Category 1 classification in the GEIS appropriate.

The staff has not identified any significant new information during its independent review of the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no offsite radiological impacts related to spent fuel and HLW disposal during the renewal term beyond those discussed in the GEIS.

- Nonradiological impacts of the uranium fuel cycle. Based on information in the GEIS, the Commission found that

The nonradiological impacts of the uranium fuel cycle resulting from the renewal of an operating license for any plant are found to be SMALL.

The staff has not identified any significant new information during its independent review of the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no nonradiological impacts of the uranium fuel cycle during the renewal term beyond those discussed in the GEIS.

- Low-level waste storage and disposal. Based on information in the GEIS, the Commission found that

The comprehensive regulatory controls that are in place and the low public doses being achieved at reactors ensure that the radiological impacts to the environment will remain small during the term of a renewed license. The maximum additional on-site land that may be required for low-level waste storage during the term of a renewed license and associated impacts will be small. Nonradiological impacts on air and water will be negligible. The radiological and nonradiological environmental impacts of long-term disposal of low-level waste from any individual plant at licensed sites are small. In addition, the Commission concludes that there is reasonable assurance that sufficient low-level waste disposal capacity will be made available when needed for facilities to be decommissioned consistent with NRC decommissioning requirements.

The staff has not identified any significant new information during its independent review of the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of low-level waste storage and disposal associated with the renewal term beyond those discussed in the GEIS.

- Mixed waste storage and disposal. Based on information in the GEIS, the Commission found that

The comprehensive regulatory controls and the facilities and procedures that are in place ensure proper handling and storage, as well as negligible doses and exposure to toxic materials for the public and the environment at all plants. License renewal will not increase the small, continuing risk to human health and the environment posed by mixed waste at all plants. The radiological and non-radiological environmental impacts of long-term disposal of mixed waste from any individual plant at licensed sites are small. In addition, the Commission concludes that there is reasonable assurance that sufficient mixed waste

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1 disposal capacity will be made available when needed for facilities to be
2 decommissioned consistent with NRC decommissioning requirements.

3
4 The staff has not identified any significant new information during its independent review of
5 the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its evaluation of
6 other available information. Therefore, the staff concludes that there are no impacts of
7 mixed waste storage and disposal associated with the renewal term beyond those
8 discussed in the GEIS.

- 9
10 • Onsite spent fuel. Based on information in the GEIS, the Commission found that

11
12 The expected increase in the volume of spent fuel from an additional 20 years of
13 operation can be safely accommodated on site with small environmental effects
14 through dry or pool storage at all plants if a permanent repository or monitored
15 retrievable storage is not available.

16
17 The staff has not identified any significant new information during its independent review of
18 the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its evaluation of
19 other available information. Therefore, the staff concludes that there are no impacts of
20 onsite spent fuel associated with license renewal beyond those discussed in the GEIS.

- 21
22 • Nonradiological waste. Based on information in the GEIS, the Commission found that

23
24 No changes to generating systems are anticipated for license renewal. Facilities
25 and procedures are in place to ensure continued proper handling and disposal at
26 all plants.

27
28 The staff has not identified any significant new information during its independent review of
29 the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its evaluation of
30 other available information. Therefore, the staff concludes that there are no nonradiological
31 waste impacts during the renewal term beyond those discussed in the GEIS.

- 32
33 • Transportation. Based on information contained in the GEIS, the Commission found
34 that

35
36 The impacts of transporting spent fuel enriched up to 5 percent uranium-235 with
37 average burnup for the peak rod to current levels approved by NRC up to
38 62,000 MWd/MTU and the cumulative impacts of transporting high-level waste to
39 a single repository, such as Yucca Mountain, Nevada are found to be consistent
40 with the impact values contained in 10 CFR 51.52(c), Summary

Table S-4--Environmental Impact of Transportation of Fuel and Waste to and from One Light-Water-Cooled Nuclear Power Reactor. If fuel enrichment or burnup conditions are not met, the applicant must submit an assessment of the implications for the environmental impact values reported in Sec. 51.52.

McGuire meets the fuel-enrichment and burnup conditions set forth in Addendum 1 to the GEIS. In recent years, licensees have requested authorization to increase fuel enrichment and fuel burnup. In its letter dated September 22, 1999 (NRC 1999b), the staff approved a maximum burnup rate of 60,000 MWd/MTU. Based on a reassessment of the impacts resulting from the transportation of spent fuel (NRC 2001), the staff's preliminary determination is that the environmental impacts at a burnup rate of 62,000 MWd/MTU are unchanged from those summarized in Table S-4. The staff has not identified any significant new information during its independent review of the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of transportation associated with license renewal beyond those discussed in the GEIS.

6.2 References

10 CFR 51. Code of Federal Regulations, Title 10, *Energy*, Part 51, "Environmental Protection Regulations for Domestic Licensing and Related Regulatory Functions."

10 CFR 54. Code of Federal Regulations, Title 10, *Energy*, Part 54, "Requirements for Renewal of Operating Licenses for Nuclear Power Plants."

10 CFR 63. Code of Federal Regulations. Title 10, *Energy*, Part 63, "Disposal of High-Level Radioactive Wastes in a Geologic Repository at Yucca Mountain, Nevada."

40 CFR 191. Code of Federal Regulations, Title 40, *Protection of Environment*, Part 191, "Environmental Radiation Protection Standards for Management and Disposal of Spent Nuclear Fuel, High-Level and Transuranic Radioactive Waste."

40 CFR 197. Code of Federal Regulations. Title 40, *Protection of Environment*, Part 197, "Public Health and Environmental Radiation Protection Standards for Yucca Mountain, Nevada."

Duke Energy Corporation (Duke). 2001. *Applicant's Environmental Report - Operating License Renewal Stage - McGuire Nuclear Station*. Charlotte, North Carolina.

Energy Policy Act of 1992. 42 USC 10101 et seq.

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1 U.S. Department of Energy (DOE). 1980. *Final Environmental Impact Statement:*
2 *Management of Commercially Generated Radioactive Waste*. DOE/EIS 00046-G, Vols. 1-3,
3 Washington, D.C.

4
5 U.S. Nuclear Regulatory Commission (NRC). 1996. *Generic Environmental Impact Statement*
6 *for License Renewal of Nuclear Plants*. NUREG-1437, Volumes 1 and 2, Washington, D.C.

7
8 U.S. Nuclear Regulatory Commission (NRC). 1999a. *Generic Environmental Impact*
9 *Statement for License Renewal of Nuclear Plants, Main Report*, "Section 6.3 - Transportation,
10 Table 9.1 Summary of findings on NEPA issues for license renewal of nuclear power plants,
11 Final Report." NUREG-1437, Volume 1, Addendum 1, Washington, D.C.

12
13 U.S. Nuclear Regulatory Commission (NRC). 1999b. Letter from F. Rinaldi, NRC, to
14 H.B. Barron, Vice President, McGuire Site, Duke Energy Corporation. Subject: McGuire
15 Nuclear Station, Units 1 and 2 Re: ISSUANCE OF AMENDMENTS.

16
17 U.S. Nuclear Regulatory Commission (NRC). 2001. *Environmental Effects of Extending Fuel*
18 *Burnup Above 60 GWd/MTU*, NUREG/CR-6703, Washington D.C.

7.0 Environmental Impacts of Decommissioning

Environmental issues associated with decommissioning, which result from continued plant operation during the renewal term, were discussed in the *Generic Environmental Impact Statement for License Renewal of Nuclear Plants* (GEIS), NUREG-1437, Volumes 1 and 2 (NRC 1996, 1999).^(a) The GEIS includes a determination of whether the analysis of the environmental issues could be applied to all plants and whether additional mitigation measures would be warranted. Issues were then assigned a Category 1 or a Category 2 designation. As set forth in the GEIS, Category 1 issues are those that meet all of the following criteria:

- (1) The environmental impacts associated with the issue have been determined to apply either to all plants or, for some issues, to plants having a specific type of cooling system or other specified plant or site characteristics.
- (2) A single significance level (i.e., SMALL, MODERATE, or LARGE) has been assigned to the impacts (except for collective offsite radiological impacts from the fuel cycle and from high-level waste and spent fuel disposal).
- (3) Mitigation of adverse impacts associated with the issue has been considered in the analysis, and it has been determined that additional plant-specific mitigation measures are likely not to be sufficiently beneficial to warrant implementation.

For issues that meet the three Category 1 criteria, no additional plant-specific analysis is required unless new and significant information is identified.

Category 2 issues are those that did not meet one or more of the criteria of Category 1, and therefore, additional plant-specific review of these issues is required. No Category 2 issues are related to decommissioning McGuire Nuclear Station, Units 1 and 2 (McGuire).

Category 1 issues in Table B-1 of 10 CFR Part 51, Subpart A, Appendix B, that are applicable to McGuire decommissioning following the renewal term are listed in Table 7-1. In its environmental report (ER) (Duke 2001), Duke Energy Corporation (Duke) stated "no new information exists for the issues that would invalidate the GEIS conclusions." The staff has not identified any significant new information during its independent review of the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts related to these issues beyond those discussed in

(a) The GEIS was originally issued in 1996. Addendum 1 to the GEIS was issued in 1999. Hereafter, all references to the "GEIS" include the GEIS and its Addendum 1.

Table 7-1. Category 1 Issues Applicable to Decommissioning of McGuire, Units 1 and 2, Following the Renewal Term

ISSUE—10 CFR Part 51, Subpart A, Appendix B, Table B-1	GEIS Section
DECOMMISSIONING	
Radiation Doses	7.3.1; 7.4
Waste Management	7.3.2; 7.4
Air Quality	7.3.3; 7.4
Water Quality	7.3.4; 7.4
Ecological Resources	7.3.5; 7.4
Socioeconomic Impacts	7.3.7; 7.4

the GEIS. For all of these issues, the staff concluded in the GEIS that the impacts are SMALL, and plant-specific mitigation measures are not likely to be sufficiently beneficial to be warranted.

A brief description of the staff's review and the GEIS conclusions, as codified in Table B-1, for each of the issues follows:

- Radiation doses. Based on information in the GEIS, the Commission found that

Doses to the public will be well below applicable regulatory standards regardless of which decommissioning method is used. Occupational doses would increase no more than 1 man-rem [0.01 person-Sv] caused by buildup of long-lived radionuclides during the license renewal term.

The staff has not identified any new and significant information during its independent review of the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no radiation doses associated with decommissioning following license renewal beyond those discussed in the GEIS.

- Waste management. Based on information in the GEIS, the Commission found that

Decommissioning at the end of a 20-year license renewal period would generate no more solid wastes than at the end of the current license term. No increase in the quantities of Class C or greater than Class C wastes would be expected.

1 The staff has not identified any new and significant information during its independent
2 review of the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its
3 evaluation of other available information. Therefore, the staff concludes that there are no
4 impacts of solid waste associated with decommissioning following the license renewal term
5 beyond those discussed in the GEIS.

- 6
7 • Air quality. Based on information in the GEIS, the Commission found that

8
9 Air quality impacts of decommissioning are expected to be negligible either at
10 the end of the current operating term or at the end of the license renewal term.

11
12 The staff has not identified any new and significant information during its independent
13 review of the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its
14 evaluation of other available information. Therefore, the staff concludes that there are no
15 impacts of license renewal on air quality during decommissioning beyond those discussed
16 in the GEIS.

- 17
18 • Water quality. Based on information in the GEIS, the Commission found that

19
20 The potential for significant water quality impacts from erosion or spills is no
21 greater whether decommissioning occurs after a 20-year license renewal period
22 or after the original 40-year operation period, and measures are readily available
23 to avoid such impacts.

24
25 The staff has not identified any new and significant information during its independent
26 review of the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its
27 evaluation of other available information. Therefore, the staff concludes that there are no
28 impacts of the license renewal term on water quality during decommissioning beyond those
29 discussed in the GEIS.

- 30
31 • Ecological Resources. Based on information in the GEIS, the Commission found that

32
33 Decommissioning after either the initial operating period or after a 20-year
34 license renewal period is not expected to have any direct ecological impacts.

35
36 The staff has not identified any new and significant information during its independent
37 review of the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its
38 evaluation of other available information. Therefore, the staff concludes that there are no
39 impacts of the license renewal term on ecological resources during decommissioning
40 beyond those discussed in the GEIS.

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- Socioeconomic Impacts. Based on information in the GEIS, the Commission found that

Decommissioning would have some short-term socioeconomic impacts. The impacts would not be increased by delaying decommissioning until the end of a 20-year relicense period, but they might be decreased by population and economic growth.

The staff has not identified any new and significant information during its independent review of the McGuire ER (Duke 2001), the staff's site visit, the scoping process, or its evaluation of other available information. Therefore, the staff concludes that there are no impacts of license renewal on the socioeconomic impacts of decommissioning beyond those discussed in the GEIS.

7.1 References

10 CFR 51. Code of Federal Regulations, Title 10, *Energy*, Part 51, "Environmental Protection Regulations for Domestic Licensing and Related Regulatory Functions."

Duke Energy Corporation (Duke). 2001. *Applicant's Environmental Report – Operating License Renewal Stage – McGuire Nuclear Station*. Charlotte, North Carolina.

U.S. Nuclear Regulatory Commission (NRC). 1996. *Generic Environmental Impact Statement for License Renewal of Nuclear Plants*. NUREG-1437, Volumes 1 and 2, Washington, D.C.

U.S. Nuclear Regulatory Commission (NRC). 1999. *Generic Environmental Impact Statement for License Renewal of Nuclear Plants, Main Report*, "Section 6.3 – Transportation, Table 9.1, Summary of findings on NEPA issues for license renewal of nuclear power plants, Final Report." NUREG-1437, Volume 1, Addendum 1, Washington, D.C.